



Wisconsin  
Electric  
POWER COMPANY

Point Beach Nuclear Plant  
6610 Nuclear Rd., Two Rivers, WI 54241

(414) 755-2321

PBL 94-0344

November 4, 1994

Document Control Desk  
U. S. NUCLEAR REGULATORY COMMISSION  
Mail Station P1-137  
Washington, DC 20555

Gentlemen:

DOCKETS 50-266 AND 50-301  
MONTHLY OPERATING REPORTS  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

Attached are monthly operating reports for Units 1 and 2 of the Point Beach Nuclear Plant for the calendar month of October, 1994.

Sincerely,

G. J. Maxfield  
PBNP Manager

djs

Attachments

cc: L. L. Smith, PSCW  
NRC Regional Administrator, Region III  
NRC Resident Inspector

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## Point Beach Shift Operational Data Summary

October

1994

Day	Unit 1							Unit 2						
	Gen	X02	X04	X08	X27	Net MWe	Avg MWe	Gen	X02	X04	X08	X27	Net MWe	Avg MWe
1	12210	495	41	1	4	11670	486	0	0	41	1	4	-46	-2
2	12270	497	41	1	4	11728	489	0	0	36	1	4	-40	-2
3	12260	497	42	2	5	11715	488	0	1	33	2	5	-40	-2
4	12270	499	42	4	5	11721	488	0	0	34	4	5	-42	-2
5	12270	498	40	3	5	11725	489	0	0	29	3	5	-37	-2
6	12270	498	37	4	4	11728	489	0	0	30	4	4	-38	-2
7	12290	499	33	4	4	11751	490	0	0	32	4	4	-40	-2
8	12370	498	36	4	4	11829	493	0	0	32	4	4	-40	-2
9	12450	499	38	4	4	11906	496	0	0	35	4	4	-43	-2
10	12400	499	30	4	4	11864	494	0	0	28	4	4	-36	-1
11	12410	499	30	3	4	11874	495	0	0	33	3	4	-40	-2
12	12380	502	34	2	5	11838	493	0	0	39	2	5	-46	-2
13	12360	499	31	1	4	11826	493	0	0	38	1	4	-43	-2
14	12390	500	31	1	5	11854	494	0	0	40	1	5	-45	-2
15	12340	498	31	1	4	11807	492	0	0	40	1	4	-44	-2
16	12350	497	29	1	4	11819	492	0	0	40	1	4	-45	-2
17	12350	499	30	1	4	11817	492	0	0	40	1	4	-45	-2
18	12340	500	29	1	5	11806	492	0	0	41	1	5	-47	-2
19	12380	501	34	0	4	11841	493	0	0	42	0	4	-46	-2
20	12340	501	36	1	5	11798	492	0	0	43	1	5	-49	-2
21	12490	503	35	1	5	11947	498	0	0	45	1	5	-51	-2
22	12260	502	35	1	4	11719	488	0	0	47	1	4	-52	-2
23	12370	502	34	1	4	11830	493	0	0	45	1	4	-50	-2
24	12370	503	37	1	5	11825	493	0	0	46	1	5	-52	-2
25	12410	505	40	1	4	11860	494	0	0	41	1	4	-46	-2
26	12400	504	38	1	6	11852	494	0	0	47	1	6	-54	-2
27	12380	504	36	1	5	11834	493	0	0	50	1	5	-56	-2
28	12390	504	34	1	5	11846	494	0	0	242	1	5	-248	-10
29	12400	504	36	1	5	11855	494	0	0	280	1	5	-285	-12
30	12930	522	36	1	5	12367	515	0	0	335	1	5	-341	-14
31	12380	506	30	1	5	11839	493	0	0	337	1	5	-343	-14

## MONTHLY TOTALS - UNIT 1

Gross generation: 383480 MWhr  
 Total station service: 16799 MWhr  
 Net generation: 366682 MWhr  
 Average daily power: 493 MWe

## MONTHLY TOTALS - UNIT 2

Gross generation: 0 MWhr  
 Total station service: 2421 MWhr  
 Net generation: -2421 MWhr  
 Average daily power: -3 MWe

# OPERATING DATA REPORT

DOCKET NO. 50-266

DATE: 11/04/94

COMPLETED BY: M. B. Koudelka

TELEPHONE: 414 755-6480

## OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT - UNIT 1 . . . . . NOTES
2. REPORTING PERIOD: October - 1994 . . . . .
3. LICENSED THERMAL POWER (MWT): 1518.5 . . . . .
4. NAMEPLATE RATING (GROSS MWE): 523.8 . . . . .
5. DESIGN ELECTRICAL RATING (NET MWE): 497.0 . . . . .
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 509.0 . . . . .
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.0 . . . . .
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:  
NA
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NA
10. REASONS FOR RESTRICTIONS, (IF ANY):  
NA

	THIS MONTH	YEAR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745.0	7,296.0	210,264.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	745.0	6,670.6	175,389.5
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	667.3
14. HOURS GENERATOR ON LINE	745.0	6,622.1	172,262.3
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	846.9
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,131,283	9,873,475	243,355,245
17. GROSS ELECTRICAL ENERGY GENERATED	383,480	3,341,200	82,195,330
18. NET ELECTRICAL ENERGY GENERATED (MWH)	366,681	3,193,397	78,348,987
19. UNIT SERVICE FACTOR	100.0%	90.8%	81.9%
20. UNIT AVAILABILITY FACTOR	100.0%	90.8%	82.3%
21. UNIT CAPACITY FACTOR (USING MDC NET)	101.5%	90.2%	76.5%
22. UNIT CAPACITY FACTOR (USING DER NET)	99.0%	88.1%	75.0%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	1.5%
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refueling Outage, 03/11/95, 39 days			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NA			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

## POINT BEACH NUCLEAR PLANT

## AVERAGE DAILY UNIT POWER LEVEL

MONTH OCTOBER - 1994

DOCKET NO. 50-266

UNIT NAME Point Beach, Unit 1

DATE November 4, 1994

COMPLETED BY M. B. Koudelka

TELEPHONE (414) 755-6480

DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET
1	486	11	495	21	498
2	489	12	493	22	488
3	488	13	493	23	493
4	488	14	494	24	493
5	489	15	492	25	494
6	489	16	492	26	494
7	490	17	492	27	493
8	493	18	492	28	494
9	496	19	493	29	494
10	494	20	492	30	515
				31	493

## POINT BEACH NUCLEAR PLANT

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OCTOBER - 1994

Docket No. 50-266  
 Unit Name Point Beach, Unit 1  
 Date November 4, 1994  
 Completed By M. B. Koudelka  
 Telephone No. 414/755-6480

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective Action To Prevent Recurrence
NA	NA	NA	NA	NA	NA	NA	NA	NA	NA

<sup>1</sup>F: Forced  
 S: Scheduled

<sup>2</sup>Reason:  
 A - Equipment Failure (explain)  
 B - Maintenance or Testing  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training &  
 Licensing Exam  
 F - Administrative  
 G - Operational Error (explain)  
 H - Other (explain)

<sup>3</sup>Method:  
 1 - Manual  
 2 - Manual Scram  
 3 - Automatic Scram  
 4 - Continuation of  
 Previous Shutdown  
 5 - Reduced Load  
 6 - Other (explain)

<sup>4</sup>Exhibit G - Instructions  
 for preparation of  
 data entry sheets  
 LER file (NUREG-0161)

<sup>5</sup>Exhibit I - Same Source

DOCKET NO. 50-266  
UNIT NAME Point Beach Unit 1  
DATE November 4, 1994  
COMPLETED BY M. B. Koudelka  
TELEPHONE 414/755-6480

Unit 1 operated at an average of 493 MWe net for the report period.

Licensee Event Reports 50-266/94-009-00, Inadvertent Emergency Diesel Start and Loss of Station Battery Charger; and, 50-266/94-010, Inadvertent Emergency Diesel Generators Start, Loss of Two Station Battery Chargers, and Unit 2 Loss of Decay Heat Removal, were submitted.

Safety-related maintenance included:

1. C02 electrical control board motor control center switches installed.
2. 1P29T turbine-driven auxiliary feedwater pump turbine bearing oils sampled.
3. 1B52-16C breaker cycled.
4. Rewired 1B52-22A breaker and cubicle.

# OPERATING DATA REPORT

DOCKET NO. 50-301

DATE: 11/04/94

COMPLETED BY: M. B. Koudelka

TELEPHONE: (414) 755-6480

## OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT - UNIT 2 . NOTES .
2. REPORTING PERIOD: October - 1994 .
3. LICENSED THERMAL POWER (MWT): 1518.5 .
4. NAMEPLATE RATING (GROSS MWE): 523.8 .
5. DESIGN ELECTRICAL RATING (NET MWE): 497.0 .
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 509.0 .
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.0 .
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:  
NA
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NA
10. REASONS FOR RESTRICTIONS, (IF ANY):  
NA

	THIS MONTH	YEAR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745.0	7,296.0	195,049.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	6,387.0	170,587.6
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	233.9
14. HOURS GENERATOR ON LINE	0.0	6,385.5	168,252.2
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	302.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	0	9,588,393	241,959,577
17. GROSS ELECTRICAL ENERGY GENERATED	0	3,243,160	82,267,420
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0	3,097,915	78,428,555
19. UNIT SERVICE FACTOR	0.0%	87.5%	86.3%
20. UNIT AVAILABILITY FACTOR	0.0%	87.5%	86.4%
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0%	87.5%	82.3%
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0%	85.4%	80.9%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	1.0%
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): NONE			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 11/01/94			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

## POINT BEACH NUCLEAR PLANT

## AVERAGE DAILY UNIT POWER LEVEL

MONTH OCTOBER - 1994DOCKET NO. 50-301UNIT NAME Point Beach, Unit 2DATE November 4, 1994COMPLETED BY M. B. KoudelkaTELEPHONE (414) 755-6480

DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET
1	-2	11	-2	21	-2
2	-2	12	-2	22	-2
3	-2	13	-2	23	-2
4	-2	14	-2	24	-2
5	-2	15	-2	25	-2
6	-2	16	-2	26	-2
7	-2	17	-2	27	-2
8	-2	18	-2	28	-10
9	-2	19	-2	29	-12
10	-1	20	-2	30	-14
				31	-14

## POINT BEACH NUCLEAR PLANT

UNIT SHUTDOWNS AND POWER REDUCTIONSREPORT MONTH OCTOBER - 1994

Docket No. 50-301  
 Unit Name Point Beach, Unit 2  
 Date November 4, 1994  
 Completed By M. B. Koudelka  
 Telephone No. 414/755-6480

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective Action Prevent Recurrence
2	9/24/94	S	745	C	1	NA	NA	NA	Scheduled refueling and maintenance outage (U2R20). Major work items include amptector upgrades, RHR pump rotor replacement, 4-rotor limit switch upgrades, main feedwater check valve replacements, reactor heat shield, SI-854A&B internals replacement, Gai-tronics upgrade, G04 emergency diesel generator tie-in work, reactor coolant pump seal water flow transmitter replacement, safeguards sequence timing relay replacements, control rod drive mechanism control current timing modification, degraded grid voltage relay work, steam generator blowdown heat exchanger replacement, and containment service water pipe supports as part of continued Bulletin 79-14 upgrades.

<sup>1</sup>F: Forced  
 S: Scheduled

<sup>2</sup>Reason:  
 A - Equipment Failure (explain)  
 B - Maintenance or Testing  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training &  
 Licensing Exam  
 F - Administrative  
 G - Operational Error (explain)  
 H - Other (explain)

<sup>3</sup>Method:  
 1 - Manual  
 2 - Manual Scram  
 3 - Automatic Scram  
 4 - Continuation of  
 Previous Shutdown  
 5 - Reduced Load  
 6 - Other (explain)

<sup>4</sup>Exhibit G - Instructions  
 for preparation of  
 data entry sheets  
 LER file (NUREG-0161)

<sup>5</sup>Exhibit I - Same Source

DOCKET NO. 50-301  
UNIT NAME Point Beach Unit 2  
DATE November 4, 1994  
COMPLETED BY M. B. Koudelka  
TELEPHONE 414/755-6480

Unit 2 was shutdown for a refueling and maintenance outage since September 24, 1994. The unit is estimated for startup on November 1, 1994.

Licensee Event Reports 50-301/94-002-01, Quarterly Technical Specifications Test of PORV Block Valve Not Performed; 50-301/94-003-00, Steam Generator Tube Degradation; and 50-301/94-004-00, Late Reactor Coolant System Chloride Sample, were submitted.

Safety-related maintenance included:

1. 2A05 bus loss of voltage relay replacements.
2. 2B52-40B&C breaker direct tripping actuator inspections.
3. 2C03 steam and turbine auxiliary control board Train A power available light socket replaced.
4. As-built wire tracing performed on panels 2C75, 2C208B, 2C208C, and 2N04.
5. Four-rotor limit switches installed on 2CV-112C-O, 2CC-754B-O, SI-896A-O, 2SI-841B-O and 2SI-860D-O.
6. 2CV-133 residual heat removal to letdown isolation valve regulator repaired.
7. 2CV-371-S letdown line containment isolation solenoid valve rebuilt.
8. 2FE-175 and 2FE-176 reactor coolant pump A and B #1 seal water return low range flow elements repaired.
9. 2FI-626 residual heat removal pump flow indicator repaired.
10. Various wiring changes made and testing performed on panels and buses in support of G04 emergency diesel generator tie-in work.

11. 2HS-22 Loop A cubicle snubber stud and nut replacement.
12. 2HX-15A8 containment accident recirculation heat exchanger welded in pipe plug.
13. 2LT-460B HX-1A steam generator wide range level transmitter calibrated.
14. Replaced breakers on 2MOB-322, 2MOB-331, 2MOB-333, 2MOB-334, and 2MOB-335 and seventeen MOBs cycled.
15. 2MS-2019-M steam generator header auxiliary feedwater pump steam supply motor-operated valve motor torque switch replacement.
16. 2MS-5916 auxiliary feedwater pump turbine casing pressure sentinel lockwire reinstalled.
17. 2N42 nuclear instrumentation power range channel module replaced and calibrated.
18. 2P1B-M reactor coolant pump motor repaired crack in weld.
19. 2P10B residual heat removal pump rotating assembly replaced.
20. 2PC-429C-XAT Train A pressurizer pressure to safety injection test switch replacement.
21. 2PI-4198A air lock outer door vacuum pump suction pressure indicator repaired.
22. 2PT-420B and 2PT-430 reactor coolant and pressurizer pressure transmitters cleaned and fittings tightened.
23. 2R1 reactor vessel head shielding installed.
24. 2RC-508-O reactor makeup water supply valve stroked.
25. 2RC-538 pressurizer relief tank gas analyzer system isolation valve diaphragm replaced.
26. 2RC-581 pressurizer and reactor vessel line low point drain valve repaired.
27. 2RH-624 and 2RH-625 residual heat removal heat exchanger outlets control position indicators installed.
28. 2RH-709B residual heat removal pump discharge valve repairs.

29. 2RH-861B residual heat removal pump suction header relief to pressurizer relief tank valve tested and setpoint adjustment.
30. 2SC-966C reactor coolant hot leg sample valve stroke tested.
31. 2SI-825A refueling water storage tank outlet to safety injection pump gasket replacement.
32. 2SI-850A&B residual heat removal pump Sump B suction valve limit switch setting adjusted.
33. 2SI-851A residual heat removal pump suction from containment sump B valve disc and seat inspection.
34. 2SI-854A refueling water storage tank to residual heat removal pump suction header check valve replacement.
35. 2SI-854B refueling water storage tank to residual heat removal suction header check valve rubber seat and disc repair.
36. 2SI-856B refueling water storage tank outlet to residual heat removal pump suction header valve inspection.
37. 2SI-866B core deluge injection line isolation status light indication moved.
38. 2SI-896B safety injection pump suction valve disc repositioned.
39. 2SI-897A&B safety injection test line return valve status light indication adjusted.
40. 2SI-957 safety injection accumulator nitrogen header vent control valve U-bolt replaced.
41. 2SW-15A-D containment recirculation heat exchanger inlet check valves inspected.
42. 2SW-2959, 2SW-2963, 2SW-2967, and 2SW-2971 containment recirculation heat exchanger outlet relief valve replaced.
43. 2SW-4301 cavity cooling coil outlet relief valve replaced.
44. 2SW-4389 auxiliary feedwater pump discharge relief valve replaced.

45. 2TC-405L, 2TC-406L, 2TC-407L and 2TC-408L delta flux controllers calibrated.
46. Recalibrated 2TE-401B, SN-1046, 2TE-403A, SN-1054, 2TE-408A and SN-1059.
47. 2VNPSE-3213-O purge exhaust fan suction control operator instrument air regulator replaced.
48. 2VNPSE-3245 purge supply fan discharge control solenoid valve replaced.
49. 2W1A1 containment accident recirculation fan field balancing.
50. WL-1728 sump A drain valve replaced.
51. 2Z42 incore thimble seal table inspection.
52. 2Z434 refueling cavity Loop A cold leg sandbox cover gasket inspection.
53. Replaced 97 panelboard breakers.
54. Nine time delay relays were replaced.
55. Diagnostic tests performed on seven main steam and safety injection valves.