



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

November 9, 1994

Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Document Control Desk

Subject: Supplemental Information Regarding the Fracture Mechanics
Evaluation of Residual Heat Removal System Heat Exchanger
Inlet and Outlet Nozzle to Shell Welds

Byron Nuclear Power Station, Units 1 and 2
NPF-37/66; NRC Docket Nos. STN 50-454/455

Braidwood Nuclear Power Station, Units 1 and 2
NPF-72/77; NRC Docket Nos. STN 50-456/457

- References:
- 1) November 8, 1994 letter, D. M. Saccomando to NRC,
Braidwood Unit 2 Flaw Evaluation Report for Residual
Heat Removal Heat Exchanger Nozzle to Shell Welds
 - 2) November 8, 1994 letter, H. D. Pontious, Jr. to NRC,
Fracture Mechanics Evaluation of Residual Heat Removal
Systems Heat Exchanger Inlet and Outlet Nozzle to Shell
Welds

In Reference 1) Commonwealth Edison Company (ComEd) committed, in part, to submit to the Nuclear Regulatory Commission (NRC) Staff finite element analyses for the Byron and Braidwood Units 1 and 2 Residual Heat Removal (RHR) Heat Exchanger (HX) nozzle to shell welds. Attachment 2 is Westinghouse Letter Report MSE-SMT-461, "Fracture Mechanics Evaluation of Indications in Byron and Braidwood Units 1 and 2 RHX Tube Side Inlet and Outlet Nozzles" which contains three dimensional (3D) finite element analyses. This report provides additional justification that the fillet weld thickness can be credited when performing limit load structural margin assessment of the nozzle to shell welds in accordance with the methodology documented in WCAP 13454, "Fracture Mechanics Evaluation, Byron and Braidwood Units 1 and 2, Residual Heat Exchanger Tube Side Inlet and Outlet Nozzles," dated August 1992 (Proprietary) (transmitted with Reference 2).

k:\nla\brded\3dfinite.wpf\1

210000

9411230067 941109
PDR ADOCK 05000454
P PDR

APD11

November 9, 1994

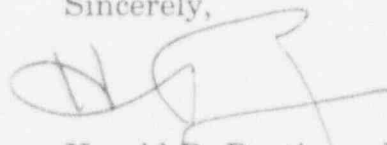
Please note that Westinghouse Letter Report MSE-SMT-461 contains information proprietary to the Westinghouse Electric Corporation as specified in the attached Westinghouse Letter NTD-NRC-94-4341 dated November 9, 1994 (Attachment 1). This letter defines how the Westinghouse Electric Corporation will comply with Title 10, Code of Federal Regulations, Part 2, Section 790 (10 CFR 2.790). Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure.

Correspondence with respect to the proprietary aspects of the items listed above should refer to Westinghouse Letter NTD-NRC-94-4341 and be addressed to N. J. Liparulo, Manager of Nuclear Safety & Regulatory Activities, Westinghouse Electric Corporation, P. O. Box 355, Pittsburgh, Pennsylvania 15230-0355.

ComEd requests that this information be included in the Staff's expedited review and approval of References 1) and 2). ComEd requests that the Staff complete their initial review and approval no later than November 10, 1994 in order to support the scheduled restart of Braidwood Station Unit 2.

Please address any questions or comments regarding this submittal to this office.

Sincerely,



Harold D. Pontious, Jr.
Nuclear Licensing Administrator

Attachment 1: Westinghouse Letter NTD-NRC-94-4341
Attachment 2: Westinghouse Letter Report MSE-SMT-461

cc: J. B. Martin, Regional Administrator - Region III
G. F. Dick, Byron Project Manager - NRR
R. R. Assa, Braidwood Project Manager - NRR
H. Peterson, Senior Resident Inspector - Byron
S. G. DuPont, Senior Resident Inspector - Braidwood
Office of Nuclear Facility Safety - IDNS

ATTACHMENT 1

WESTINGHOUSE LETTER NTD-NRC-94-4341

RHX FRACTURE MECHANICS

November 9, 1994