

NRC FORM 366  
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

## EXHIBIT A

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 NY IPS 2' 0-000000-00034111114

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

CON'T

## REPORT

CON'T

0	1
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REPORT SOURCE

L	6	0	5	0	0	0	2	4	7	7	0	2	1	3	8	3	8	0	7	1	1	8	3	9
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DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

02 During a plant start-up, measured reactor coolant system leakage in-

03 creased to 7.4 GPM. A plant shutdown commenced. Tech Spec. 3.1.F.2.C.(1)

04 limits identified leakage to 10 GPM. A containment entry team found RHR

05 valve 731 packing leaking through a controlled leakoff collection system

06 to the Reactor Coolant Drain Tank (RCDT). Valve leakage reached 14 GPM.

07 Leakage was directed to the RCDT and Waste Disposal System. Public health

08 and safety were unaffected. Similar events: R.O.76-2-1 and 76-2-19.

2 8 9 COMP VALVE 80

7 8 9		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE		COMP SUBCODE		VALVE SUBCODE	
0 9		C F 11		D 12		Z 13		V A L V E X 14		E 15		D 16	
7 8		9 10		11 12		12 13		13 14		15 16		17 18	
17		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.			
LER/RO REPORT NUMBER		8 3 21 22		0 0 3 24 26		0 1 28 29		X 30 31		J 32			
ACTION TAKEN		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER	
B 18 Z 19		A 20		A 21		0 0 3 0 37 40		Y 23		Y 24		N 25	
33 34		35 36		37 38		39 40		41 42		43 44		45 46	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)													

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS	
10	The cause of the leakage was failed valve gland packing. The valve is a
11	14" motor operated Copes Vulcan Gate Valve (DWG E-1-133420). The valve
12	was repacked and satisfactorily tested.

1	3	
1	4	

7 8 9  
FACILITY STATUS (28) 1 5 C  
% POWER 0 0 0 (29) NA  
OTHER STATUS (30)  
METHOD OF DISCOVERY (31) B Visual Observation  
DISCOVERY DESCRIPTION (32)

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 Z 33 Z 34 NA NA

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	37	Z	38	NA	39

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
1 R	10 10 10 (40)	NA	22

LOSS OF OR DAMAGE TO FACILITY (43)  
TYPE DESCRIPTION NA

7 8 9 PUBLICITY (45) NRC USE ONLY  
ISSUED DESCRIPTION (44) 2 0 [N] 44 68 69 80

NRC USE ONLY

NAME OF PREPARER Gary Hinrichs

PHONE: 914/526-5543

8307220280 830711  
PDR ADCK 05000247  
S PDR

ATTACHMENT

Docket 50-247  
LER 83-003/01X-1

Consolidated Edison Co. of NY, Inc.  
Indian Point Station, Unit 2

With the reactor critical at zero power, during the plant start-up on Sunday, February 13, 1983, the Reactor Operators identified an increase in Reactor Coolant System leakage. The leakage through the Reactor Coolant Drain Tank was approximately 3 GPM at the beginning of the day shift and had increased to approximately 7.4 GPM in about two hours. At this time a reactor shutdown and cooldown was commenced. Technical Specification 3.1.F.2.C (1) limits identified reactor coolant system leakage to 10 GPM. The leakage reached a maximum of 14 GPM.

A containment entry was made when the reactor became sub-critical. The containment entry team found the valve packing leak-off line of RHR valve 731 was extremely hot. The team was able to verify flow through this line. All leakage was directed to the Reactor Coolant Drain Tank and Waste Disposal System.

When conditions allowed the RHR system to be placed into service valve 731 was opened and backseated, thereby stopping the leakage.

The failure is attributed to the method of packing installation. Maintenance practices are being reviewed and revised. Valve 731 is a 14" motor operated gate valve manufactured by Copes Vulcan (DWG E-1-133420). The vendor was contacted and it was verified that the packing material used meets the vendor's specification. The packing material used in valve 731 is a combination of graphite impregnated braid and grapfoil rings. The valve was repacked while on its backseat. The post maintenance test included stroking the valve and hydrotesting to 2235 PSIG. This was performed satisfactorily.

John D. O'Toole  
Vice President

Consolidated Edison Company of New York, Inc.  
4 Irving Place, New York, NY 10003  
Telephone (212) 460-2533

July 11, 1983

RE: Indian Point Unit No. 2  
Docket No. 50-247  
LER-83-003/01X-1

Dr. Thomas E. Murley  
Regional Administrator - Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

Dear Dr. Murley:

Transmitted herewith is an updated report for Licensee  
Event Report LER-83-003.

Very truly yours,

*John D. O'Toole*

Attach/  
Copies to:

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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U.S. Nuclear Regulatory Commission  
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Buchanan, NY 10511

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