

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 T N S N P 2 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 32

CON'T

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REPORT SOURCE

L	6	0	5	0	0	0	3	2	8	7	0	6	1	0	8	3	8	0	7	0	8	8	3	9
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60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 Unit 2 in mode 1 with RCS temperature and pressure at 558 degrees F and 2215 psig.

On 06/10/83, steam generator blowdown containment isolation valve 2-FCV-1-32 was

04 declared inoperable. The valve failed closed and would not reopen. This event

05 [] caused entry into LCO 3.6.3.1. There was no effect on public health or safety.

06 Previous occurrences - four (SQRO-50-328/82009, -327/82054, 82066, and 82075).

07

08 | _____

09		SYSTEM CODE H I 11		CAUSE CODE E 12		CAUSE SUBCODE X 13		COMPONENT CODE V A L V O P 14				COMP. SUBCODE D 15		VALVE SUBCODE Z 16																	
7 8		9 10		11 12		13 14		15 16 17 18				19 20		21 22																	
17		EVENT YEAR 8 3 21 22		SEQUENTIAL REPORT NO. 0 8 6 24 26		OCCURRENCE CODE 0 3 28 29		REPORT TYPE L 30		REVISION NO. 0 32		ACTION TAKEN A 33		FUTURE ACTION Z 34		EFFECT ON PLANT Z 35		SHUTDOWN METHOD Z 36		HOURS 0 0 0 0 37 40				ATTACHMENT SUBMITTED N 41		NPD-4 FORM SUB. N 42		PRIME COMP. SUPPLIER N 43		COMPONENT MANUFACTURER M 1 2 44	
17		18 19		20 21		22 23		24 25		26 27		28 29		30 31		32 33		34 35		36 37		38 39		40 41		42 43		44 45			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0
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 The action of LCO 3.6.3.1.b was complied with. The valve operator diaphragm

11 (Masoneilan Model No. 38-70521-5) was found to be ruptured and the cause was

1 2 | determined to be the high temperature environment surrounding the operator. A

13 | higher temperature resistant silicone-based diaphragm was installed and the

valve was returned to service on 06/10/83. No further action is planned.

7 8 9
FACILITY STATUS (1) 5 (F) (28) % POWER (0) 4 (1) (29) OTHER STATUS (30) NA METHOD OF DISCOVERY (B) (31) DISCOVERY DESCRIPTION (32) Operator observation
7 8 9 10 11 12 13 14 15 16
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) NA LOCATION OF RELEASE (36) NA
7 8 9 10 11 12 13 14 15 16
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (37) (38) NA
7 8 9 10 11 12 13 14 15 16
PERSONNEL INJURIES NUMBER DESCRIPTION (40) NA
7 8 9 10 11 12 13 14 15 16
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (42) NA
7 8 9 10 11 12 13 14 15 16
PUBLICITY ISSUED DESCRIPTION (44) NA
7 8 9 10 11 12 13 14 15 16
NRC USE ONLY

NRC USE ONLY

USNRC
ATLANTA, GEORGIA
TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

July 8, 1983 3 JUL 14 All: 08

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

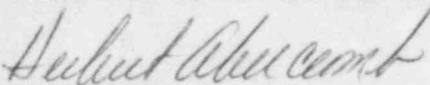
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 2 - DOCKET
NO. 50-328 - FACILITY OPERATING LICENSE DPR-79 - REPORTABLE OCCURRENCE
REPORT SQRO-50-328/83086

The enclosed report provides details concerning the inoperability of a
steam generator blowdown containment isolation valve. This report is
submitted in accordance with Sequoyah unit 2 Technical Specification
6.9.1.13.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



f H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

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Washington, D.C. 20555

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Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Sequoyah