



MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

July 19, 1983

NUCLEAR PRODUCTION DEPARTMENT

U. S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Washington, D. C. 20555

Attention: Mr. Harold R. Denton, Director

Dear Mr. Denton:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-13
File 0260/L-860.0/15180
Additional Information to
Support Changes to the
Grand Gulf Technical
Specifications
AECM-83/0407

On June 30, 1983, Mississippi Power & Light (MP&L) Company representatives met with members of the Nuclear Regulatory Commission (NRC) staff to discuss changes to the Grand Gulf Nuclear Station, Unit One Technical Specifications.

As a result of this meeting, MP&L agreed to supply additional information to support certain change requests which are, as yet, not approved. Attached, you will find additional information to support this commitment.

Yours truly,

L. F. Dale
Manager of Nuclear Services

JOF/SHH/JDR:sap
Attachments

cc: (See Next Page)

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MISSISSIPPI POWER & LIGHT COMPANY

cc: Mr. J. B. Richard (w/o)
Mr. R. B. McGehee (w/o)
Mr. T. B. Conner (w/o)
Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a)
Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. J. P. O'Reilly, Regional Administrator (w/a)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II
101 Marietta St., N.W., Suite 2900
Atlanta, Georgia 30303

PACKAGE 4, ITEM #8 (PCOL 83-08)

Item #8 (GGNS - 589) requested that the tolerance associated with the Relief Function of the Safety Relief Valves be changed from $\pm 1\%$ to ± 15 psi. The NRC Staff requested additional justification.

Grand Gulf takes credit for both the relief mode and the safety mode of the combination safety relief valves in satisfying the overpressurization protection requirements of the reactor coolant pressure boundary. This pressure relief system is in conformance with Commission regulations, applicable regulatory guides, and industry standards as noted in the GGNS SER (NUREG-0831) Section 5.2.2.

Article NB-7614.3 of Section III of the ASME Boiler and Pressure Vessel Code states the popping point tolerance shall not exceed $\pm 1\%$ of the set pressure "unless a greater tolerance is established as permissible in the Overpressure Protection Report and noted in the Design Specification."

The Nuclear Boiler System Design Specification states the Technical Specification limits for the relief pressure sensor loops should be 1118/1128/1138 psig based on 1103/1113/1123 psig nominal setpoints, ± 15 psi drift allowance, and 1133/1143/1153 psig analytical design limits. These values are consistent with and permitted by the Reactor Vessel Overpressure Protection Report (The Overprotection Report is included as Appendix 5A in the FSAR. Section 5A4.5 discusses the setpoint tolerance assumptions). The analysis was performed assuming setpoints in the range of 1145 to 1175 psig.