

OPERATING DATA REPORT

DOCKET NO. 50 - 277

DATE JULY 13, 1983

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR
TELEPHONE (215) 841-5022

OPERATING STATUS

1. UNIT NAME: PEACH BOTTOM UNIT 2
2. REPORTING PERIOD: JUNE, 1983
3. LICENSED THERMAL POWER (MWT): 3293
4. NAMEPLATE RATING (GROSS MWE): 1152
5. DESIGN ELECTRICAL RATING (NET MWE): 1065
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1098
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1051

NOTES: UNIT 2 EXPERIENCED ONE
FORCED LOAD REDUCTION
AND ONE SCHEDULED LOAD
REDUCTION.

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):
10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720	4,343	78,791
12. NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	3,824.8	58,914.5
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	720.0	3,743.8	57,292.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	2,326,630	11,912,451	168,421,423
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	769,140	3,932,330	55,452,840
18. NET ELECTRICAL ENERGY GENERATED (MWH)	742,485	3,792,982	53,178,131
19. UNIT SERVICE FACTOR	100.0	86.2	72.7
20. UNIT AVAILABILITY FACTOR	100.0	86.2	72.7
21. UNIT CAPACITY FACTOR (USING MDC NET)	98.1	83.1	64.2
22. UNIT CAPACITY FACTOR (USING DER NET)	96.8	82.0	63.4
23. UNIT FORCED OUTAGE RATE	0.0	9.3	7.8

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
REFUELING-MAINTENANCE OUTAGE BEGINNING 10/15/83 FOR 13 WEEKS.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED
INITIAL CRITICALITY	-----	-----
INITIAL ELECTRICITY	-----	-----
COMMERCIAL OPERATION	-----	-----

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ENGINEER-IN-CHARGE

LICENSING SECTION

GENERATION DIVISION-NUCLEAR

TELEPHONE (215) 841-5022

OPERATING STATUS

1. UNIT NAME: PEACH BOTTOM UNIT 3

2. REPORTING PERIOD: JUNE, 1983

3. LICENSED THERMAL POWER (MWT): 3293

4. NAMEPLATE RATING (GROSS MWE): 1152

5. DESIGN ELECTRICAL RATING (NET MWE): 1065

6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1098

7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1035

NOTES: UNIT 3 CONTINUED SCHEDULED

SHUTDOWN FOR REFUELING.

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):

10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720	4,343	74,687
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0	1,039.3	54,969.0
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	0.0	1,026.3	53,627.1
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0	2,742,355	155,957,983
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0	894,560	51,135,090
18. NET ELECTRICAL ENERGY GENERATED (MWH)	* -6,374	825,998	49,068,790
19. UNIT SERVICE FACTOR	0.0	23.6	71.8
20. UNIT AVAILABILITY FACTOR	0.0	23.6	71.8
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	18.4	63.5
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	17.9	61.7
23. UNIT FORCED OUTAGE RATE	0.0	2.6	7.3

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 SCHEDULED SHUTDOWN FOR REFUELING AND MAINTENANCE,
 STARTED 2/13/83.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 8/15/83

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED
INITIAL CRITICALITY	-----	-----
INITIAL ELECTRICITY	-----	-----
COMMERCIAL OPERATION	-----	-----

* - NEGATIVE VALUE REPORTED FOR CONSISTENCY WITH FEDERAL ENERGY REGULATORY COMMISSION REPORTS.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50 - 277

UNIT NAME PEACH BOTTOM UNIT 2

DATE JULY 14, 1983

REPORT MONTH JUNE, 1983

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.H. ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR
TELEPHONE (215) 841-5022

NO.	DATE	TYPE (1)	DURATION (HOURS)	REASON (2)	METHOD OF SHUTTING DOWN (3)	LICENSEE EVENT REPORT #	SYSTEM COMPONENT		CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
							CODE (4)	CODE (5)	
16	830615	P	0.0	H	4	NA	CC	DEHINX	LOAD REDUCTION IN ORDER TO REDUCE THE MAIN STEAM LINE RADIATION LEVEL.
17	830618	S	0.0	H	4	NA	RC	222222	LOAD REDUCTION FOR CONTROL ROD PATTERN ADJUSTMENT.

(1)

P - FORCED
S - SCHEDULED

(2)

REASON
A - EQUIPMENT FAILURE (EXPLAIN)
B - MAINTENANCE OR TEST
C - REFUELING
D - REGULATOR RESTRICTION
E - OPERATOR TRAINING + LICENSE EXAMINATION
F - ADMINISTRATIVE
G - OPERATIONAL ERROR (EXPLAIN)
H - OTHER (EXPLAIN)

(3)

METHOD
1 - MANUAL
2 - MANUAL SCRAM.
3 - AUTOMATIC SCRAM.
4 - OTHER (EXPLAIN)

(4)

EXHIBIT G - INSTRUCTIONS
FOR PREPARATION OF DATA
ENTRY SHEETS FOR LICENSE
EVENT REPORT (LER)
FILE (NUREG-0161)

(5)

EXHIBIT I - SAME SOURCE

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50 - 278

UNIT NAME PEACH BOTTOM UNIT 3

DATE JULY 13, 1983

REPORT MONTH JUNE, 1983

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.S. ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR
TELEPHONE (215) 841-5527

NO.	DATE	TYPE (1)	DURATION (HOURS) (2)	REASON (3)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
3	830601	S	720.0 720.0	C	1	NA	RC	FUELIX	CONTINUING REFUELING OUTAGE.

(1)

(2)

(3)

(4)

F - FORCED
S - SCHEDULED

REASON

A - EQUIPMENT FAILURE (EXPLAIN)
B - MAINTENANCE OR TEST
C - REFUELING
D - REGULATOR RESTRICTION
E - OPERATOR TRAINING + LICENSE EXAMINATION
F - ADMINISTRATIVE
G - OPERATIONAL ERROR (EXPLAIN)
H - OTHER (EXPLAIN)

METHOD

1 - MANUAL
2 - MANUAL SCRAM.
3 - AUTOMATIC SCRAM.
4 - OTHER (EXPLAIN)

EXHIBIT G - INSTRUCTIONS
FOR PREPARATION OF DATA
ENTRY SHEETS FOR LICENSEE
EVENT REPORT (LER)
FILE (NUREG-0161)

(5)

EXHIBIT I - SAME SOURCE

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50 - 277

UNIT PEACH BOTTOM UNIT 2

DATE JULY 13 , 1983

COMPANY PHILADELPHIA ELECTRIC COMPANY

W.M.ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR

TELEPHONE (215) 841-5022

MONTH JUNE 1983

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	1047	17	1016
2	1053	18	994
3	1053	19	846
4	1050	20	1032
5	1044	21	1044
6	1056	22	1046
7	1052	23	1045
8	1054	24	1053
9	1049	25	1043
10	1049	26	1045
11	1039	27	1045
12	1039	28	1047
13	1034	29	1043
14	1030	30	1041
15	938		
16	1011		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50 - 278

UNIT PEACH BOTTOM UNIT 3

DATE JULY 13 , 1983

COMPANY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR

TELEPHONE (215) 841-5022

MONTH JUNE 1983

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0		
16	0		

REFUELING INFORMATION

1. Name of facility:

Peach Bottom Unit 2

2. Scheduled date for next refueling shutdown:

October 15, 1983

3. Scheduled date for restart following refueling:

January 14, 1984

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes

If answer is yes, what, in general, will these be?

Technical Specifications to accommodate reload fuel
Modifications to reactor core operating limits are
expected.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

September 30, 1983

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

None expected

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

(a) Core - 764 Fuel Assemblies

(b) Fuel Pool - 1170 Fuel Assemblies, 58 Fuel Rods

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The spent fuel pool storage capacity has been relicensed for 2816 fuel assemblies.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

March, 1990 (September, 1985 with reserve for full core discharge)

REFUELING INFORMATION

1. Name of facility:

Peach Bottom Unit 3

2. Scheduled date for next refueling shutdown:

February 13, 1983

3. Scheduled date for restart following refueling:

August 15, 1983

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes

If answer is yes, what, in general, will these be?

Technical Specifications to accommodate reload fuel.
Modifications to reactor core operating limits are expected.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

Submitted on December 30, 1982, amended by NRC on May 4, 1983

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, now operating procedures:

None expected

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

(a) Core - 764 Fuel Assemblies

(b) Fuel Pool - 1212 Fuel Assemblies, 6 Fuel Rods

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The spent fuel pool storage capacity has been relicensed for 2816 fuel assemblies.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

September, 1990 (March, 1986 with reserve for fuel core discharge)

Peach Bottom Atomic Power Station
Narrative Summary of Operating Experiences
June, 1983

Unit 2

The unit began the month at full power. On June 10, a Reactor Protection System instrument power supply was replaced when high ripple voltage was detected during routine Surveillance Tests. On June 11 the Reactor Water Cleanup System Inlet Inboard Isolation Valve failed to close during Surveillance Test. The valve was declared operable on June 13 after the problem was identified to be auxilliary switch contacts at the motor contactor, and a method was employed to ensure that the contacts would be verified each time the valve was opened. Shortly after the 'G' Condensate Demineralizer was put in service on June 15, the main steam line radiation increased to 150% of full power background. The demineralizer was removed from service and reactor power reduced to 810 MWe to stabilize main steam line radiation levels. The unit returned to full power operation early on June 16. Reactor power was reduced to 700 MWe on June 18 for a control rod adjustment. The unit was returned to full power on June 20. On the evening of June 20, it was discovered that the relief device on one phase of a start-up source transformer had failed. This startup source was de-energized until this relief device was repaired early on June 21. On June 24, during Surveillance Testing, the High Pressure Coolant Injection System failed to operate. The cause of the failure was found to be a failed electronic component which was replaced and the system tested satisfactorily. On June 28, the annual Emergency Plan Graded Exercise was conducted. Results were satisfactory, with minor deficiencies noted. The unit ended the month at full power.

Unit 3

Unit 3's Fifth Refueling Outage continued throughout the month. Reactor refueling was completed on June 20. Major activities ongoing throughout the month included clad overlay repair to Reactor Recirculation System Piping and Shutdown Cooling Piping, maintenance and testing of various Primary Containment System Valves, and mud removal from the Service Water Bays.

Summary of 10CFR50.59 Review for Temporary On-Site
Storage of Radioactive Material

A storage area will be established on the second floor of the former Unit 1 auxiliary building for the temporary storage of radioactive material utilized at the Peach Bottom Unit 2-3 facility. The safety evaluation, performed in accordance with Generic Letter 81-38, concluded that storage in the designated area as specified in the evaluation meets the regulatory requirements and does not constitute an unreviewed safety question. The amount of radioactive material present will be controlled based on the total volume and measured dose rate limitations specified in the safety evaluation. Only dry material in metal containers will be stored at the storage facility. Ten percent of the total storage volume (720 cubic feet) may have contact dose rates up to 50 mrem/hr. The remainder of the material (6480 cubic feet) will have an average contact dose rate per container of less than 10 mrem/hr. The dose assessment indicates that the storage facility is within the regulatory limitations of 40CFR190 and 10CFR20. A chain link fence surrounds the former Unit 1 facility with access under the control of the Peach Bottom security organization. This information is being provided as requested in a June 12, 1980 letter (R.W. Reid, NRC to E.G. Bauer, Jr., PECO)

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

July 14, 1983

Docket Nos. 50-277
50-278

Director
Office of Inspection & Enforcement
US Nuclear Regulatory Commission
Washington, DC 20555

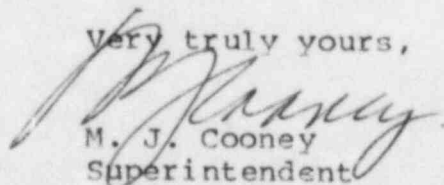
Attention: Document Control Desk

SUBJECT: Peach Bottom Atomic Power Station
Monthly Operating Report

Gentlemen:

Attached are twelve copies of the monthly operating report for Peach Bottom Units 2 and 3 for the month of June, 1983 forwarded pursuant to Technical Specification 6.9.1.C under the guidance of Regulatory Guide 10.1, Revision 4.

Very truly yours,



M. J. Cooney
Superintendent
Nuclear Generation Division

Attachment

cc: Dr. T. E. Murley, NRC
Mr. A. R. Blough, NRC Site Inspector
Mr. Stan P. Mangi, Dept. of Envir. Resources
Mr. P. A. Ross, NRC
INPO Records Center

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