

CONTROL BLOCK:

--	--	--	--	--	--

 (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

09		SYSTEM CODE S F 11		CAUSE CODE E 12		CAUSE SUBCODE B 13		COMPONENT CODE I N S T R U 14		COMP SUBCODE S 15		VALVE SUBCODE Z 16	
7 8		9 10		11		12		13 18		19		20	
17		EVENT YEAR 8 3 21 22		SEQUENTIAL REPORT NO. 0 4 8 24 26		OCCURRENCE CODE 0 3 28 29		REPORT TYPE L 30		REVISION NO. 0 32			
ACTION TAKEN E 18		FUTURE ACTION Z 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 22 40		ATTACHMENT SUBMITTED N 23		NPRD-4 FORM SUB. Y 24	
33 34		35 36		37 40		41		42		43		44 47	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27												PRIME COMP. SUPPLIER N 25	
												COMPONENT MANUFACTURER Y 0 1 1 0 26	

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				
1	5	E	28	Q	9	2	29	N/A	B	31	Routine Surveillance	32
ACTIVITY CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE						
1	6	Z	33	Z	34	N/A	35	N/A	36			

NUMBER		DESCRIPTION	
1	2	40	N/A

8307200191 8307200192

PUBLICITY		DESCRIPTION (45)		NRC USE ONLY									
2	0	N	(44) N/A										

PHONE (815) 942-2920 x526

IE 22



Commonwealth Edison

DEVIATION REPORT

DVR NO. 12 - 2 - 83 - 88
STA UNIT YEAR NO.

PART 1 TITLE OF DEVIATION

OCCURRED

U-2 Rx Level Low Level ECCS Initiate Switch

6/16/83

1330

DATE

TIME

SYSTEM AFFECTED 263

PLANT STATUS AT TIME OF EVENT

R* Press Vessel Inst.

MODE Run

2325.88

PWR(MWT)

744.45

LOAD(MWE)

TESTING

☒ YES ☐ NO

DESCRIPTION OF EVENT

Level indicating switch 2-263-72B switch (7-8) to relay 1430-105B

tripped at 111.3 inches differential water column increasing. Limit is 111.7

inches differential water column increasing. This puts ECCS core spray II and

auto blowdown initiation at greater than 88 inches vessel level decreasing.

(88 inches = 111.7 inches differential water column). Readjusted to 112.5 inches

differential water column increasing and verified repeatability.

10 CFR50.72 NRC RED PHONE

☐☒

NOTIFICATION MADE

YES

NO

EQUIPMENT FAILURE N/A

☐ YES☒ NO

WORK REQUEST NO.

RESPONSIBLE SUPERVISOR

Alan R. Zapotocky

DATE

6-16-83

PART 2 OPERATING ENGINEER'S COMMENTS

The other level switches associated with this circuit were found to be

set within specifications and would have provided proper system actuation if

required.

☐ EVENT OF PUBLIC INTEREST☐ TECH. SPEC. VIOLATION☐ NON REPORTABLE OCCURRENCE☐ 14 DAY REPORTABLE/T.S.☒ 30 DAY REPORTABLE/T.S. 6.6.B.2.a☐ ANNUAL/SPECL REPORT REQ'D☐ 24-HOUR NRC NOTIFICATION REQ'D

TELEPH N/A

REGION III

DATE

TIME

TELEGM/TELECOPY N/A

REGION III

DATE

TIME

☐ CECO CORPORATE NOTIFICATION MADE
IF ABOVE NOTIFICATION IS PER 10CFR21☐ 5-DAY WRITTEN REPORT REQ'D PER 10CFR21

Telecopy

Dennis P. Galle

6/20/83

1152

TELEPH

CECO CORPORATE OFFICER

DATE

TIME

PRELIMINARY REPORT
COMPLETED AND REVIEWED

John M. Almer

OPERATING ENGINEER

6/18/83

DATE

INVESTIGATED REPORT & RESOLUTION
ACCEPTED BY STATION REVIEWRESOLUTION APPROVED AND
AUTHORIZED FOR DISTRIBUTION

STATION SUPERINTENDENT

DATE



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

July 12, 1983

DJS Ltr #83-677

James G. Keppler, Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #83-48/03L-0, Docket #050-237 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(a), reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U.S. NRC, Document Management Branch
File/NRC

JUL 14 1983

IE 22
1/1