

METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER & LIGHT COMPANY
AND
PENNSYLVANIA ELECTRIC COMPANY
THREE MILE ISLAND NUCLEAR STATION, UNIT 1

Operating License No. DPR-50
Docket No. 50-289
Technical Specification Change Request No. 126

This Technical Specification Change Request is submitted in support of Licensee's request to change Appendix A to Operating License No. DPR-50 for Three Mile Island Nuclear Station, Unit 1. As a part of this request, proposed replacement pages for Appendix A are also included.

GPU NUCLEAR CORPORATION

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By H. S. Hukill
Director, TMI-1

Sworn and subscribed
to before me this 11th
day of July, 1983.

Darla Jean Berry
Notary Public
DARLA JEAN BERRY, NOTARY PUBLIC
MIDDLETOWN BORO. DAUPHIN COUNTY
MY COMMISSION EXPIRES JUNE 17, 1985
Member, Pennsylvania Association of Notaries

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

IN THE MATTER OF

DOCKET NO. 50-289
LICENSE NO. DPR-50

GPU NUCLEAR CORPORATION

This is to certify that a copy of Technical Specification Change Request No. 126 to Appendix A of the Operating License for Three Mile Island Nuclear Station Unit 1, has, on the date given below, been filed with the U. S. Nuclear Regulatory Commission and been served on the chief executives of Londonderry Township, Dauphin County, Pennsylvania; Dauphin County, Pennsylvania; and the Pennsylvania Department of Environmental Resources, Bureau of Radiation Protection, by deposit in the United States mail, addressed as follows:

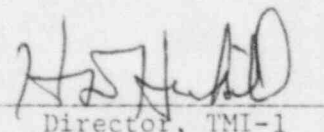
Mr. Jay H. Kopp, Chairman
Board of Supervisors of
Londonderry Township
R. D. #1, Geyers Church Road
Middletown, PA 17057

Mr. John E. Minnich, Chairman
Board of County Commissioners
of Dauphin County
Dauphin County Courthouse
Harrisburg, PA 17120

Mr. Thomas Gerusky, Director
Penna. Dept. of Environmental Resources
Bureau of Radiation Protection
P.O. Box 2063
Harrisburg, PA 17120

GPU NUCLEAR CORPORATION

BY


Director, TMI-1

DATED: July 11, 1983

I. Technical Specification Change Request No. 126

The Licensee requests that the attached changed pages replace the following pages of the existing Technical Specifications:

Page 4-1
Pages 6-8 and 6-9

II. Reason for Change Request

A - Page 4-1, Surveillance Standards (4.)

Several surveillance requirements do not specify the reactor operating conditions (modes) for which the surveillance requirement must be performed. By adding general applicability requirements in the surveillance standards, potential conflicts due to interpretations of performance of individual requirements will be avoided. This change is especially valuable when the plant is in a long term shutdown mode. This is consistent with the philosophy of the Standard Technical Specifications.

B - Pages 6-8 and 6-9, Independent Onsite Safety Review Group (ISORG)
(6.5.4)

The proposed wording changes to the Independent Onsite Safety Review Group section have been made to clarify the structure and the functions of the group, and to redefine the member engineers' qualification requirements. The use of the term "Nuclear Safety" to replace "Safety Review" emphasizes the staff concerns for technical integrity and safety of the nuclear plant.

III. Safety Evaluation Justifying Change

The surveillance change defines the applicability for requirements by relating the performance of individual surveillances to the corresponding "Limiting Condition for Operation", unless otherwise defined. Since the change specifies that surveillance requirements will be performed prior to entering, and during, reactor operational conditions (modes) for which the system/component must be operable, normal operating practices will insure compliance with the technical specifications operability requirements.

The ISORG change provides additional clarification to the existing section requirements, and does not decrease the work scope or individual qualifications for this group. It is the licensee's intent to maintain the number of IOSRG staff at one (1) manager and four (4) technical professionals. We have purposely removed reference to a minimum number to permit staffing changes when appropriate. Such flexibility is required in order to maintain the attractiveness of these positions to the most highly qualified.

Therefore these changes do not constitute unreviewed safety questions.

IV. Significant Hazards Considerations Evaluation

The operation of TMI-1 in accordance with this Tech Spec Change Request would not:

- a) involve a significant increase in the probability of occurrence or the consequences of an accident previously evaluated since this change adds requirement clarification, thereby enhancing safety.
- b) create the possibility for an accident or malfunction of a new or different kind of accident from any accident previously evaluated.
- c) involve a significant reduction in the margin of safety since they are only administrative. Therefore, no unreviewed safety questions or significant safety hazards are associated with this change.

V. Amendment Class (10 CFR 170)

The Licensee has determined that these changes are administrative in nature and involve no environmental or safety issues, therefore, a check for \$1,200 is enclosed.

VI. Implementation

Because of the nature of the change, it is requested that the implementation of the resulting amendment occur immediately upon receipt.