QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

AUGUST 1983

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TABLE OF CONTENTS

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The condenser cooling method is a closed cycle spray canal, and the Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively, pursuant to Docket Numbers 50-254 and 50-265. The date of initial reactor criticalities for Units 1 and 2 respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit 1 and March 10, 1973 for Unit 2.

This report was compiled by Becky Brown and Alex Misak, telephone number 309-654-2241, extensions 127 and 194.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

August 1-10: Unit One began the month increasing load normally from a load drop due to low Service Water pressure caused by high Spray Canal temperature. This load increase was terminated on August 2, at 1630 hours, at 770 MWe due to the 1D1 Feedwater Heater being out of service. On August 4, at 1110 hours, the unit dropped load to minimum Recirculation pump speed (430 MWe) due to low Service Water pressure. Between 1240 hours on August 5 and 0025 hours on August 6, the unit operated an open cycle cooling to reduce the Suppression Chamber temperature and increase Service Water pressure. This load was maintained until 1300 hours on August 10, when a normal load increase was initiated.

August 11-15: Unit One continued increasing load until 1950 hours on August 12, when the unit dropped load from 720 MWe to 680 MWe in order to maintain Main Condenser Vacuum. The apparent cause of the vacuum leak was discovered and remedied, and a normal load increase was initiated at 2010 hours on August 12. At 0005 hours, on August 13, the Main Condenser Vacuum leak reappeared, and the unit dropped load to 395 MWe. The unit maintained this load until 1320 hours on August 14, when the Generator was taken off line, and the Reactor mode switch was placed in HOT STAND-BY for identification and repair of the vacuum leak.

August 16-21: Unit One continued operating in Hot Stand-by until 1722 hours on August 16 when the unit was placed on line. The unit increased load to 300 MWe and maintained that load until 1900 hours on August 18 when the unit dropped load to return an Off-Gas Recombiner to service following repair of the Main Condenser Vacuum leak at 1800 hours. At 2300 hours, the unit began increasing load normally to 400 MWe. The unit maintained this load, as requested by the Load Dispatcher, unitl 0915 hours on August 19 when the unit began a normal load increase. Also, at 1035 hours, the unit began operating on open cycle cooling to reduce the Suppression Chamber temperature and increase Service Water pressure. The load increase continued until 0315 hours on August 20, when the unit held load at 670 MWe as requested by the Load Dispatcher. At 0420 hours, open cycle cooling was terminated, and at 1700 hours, the unit dropped load to 400 MWe due to low Service Water pressure.

August 22-31: At 1030 hours, on August 22, Unit One began increasing load normally from 400 MWe. At 1135 hours, open cycle cooling commenced and continued until 1755 hours. At 1635 hours, the load increase stopped with the unit at 675 MWe. This load was maintained until 1920 hours when the unit dropped load to 500 MWe. The unit maintained a load of 500 MWe until 1100 hours on August 25 when the unit increased load to 600 MWe. At 0915 hours, on August 29, the unit began increasing load normally from 600 MWe

Unit One (Continued)

to full power. The unit operated on open cycle cooling from 0930 hours to 2200 hours. At 2100 hours, the normal load increase was terminated, and the unit dropped load from 750 MWe to 600 MWe. At 0000 hours, on August 31, the unit dropped load to 377 MWe for MSIV testing. Following the testing, at 0515 hours, a normal load increase to 600 MWe was initiated. The unit achieved a load of 600 MWe at 0830 hours and maintained that load for the remainder of the month.

B. Unit Two

Unit Two operated throughout the month derated due to End of Cycle fuel depletion.

August 1-9: At 0815 hours, on August 1, the unit increased load from 325 MWe to 425 MWe as requested by the Load Dispatcher. The unit maintained this load until 0115 hours on August 3 when the unit dropped load to 295 MWe as requested by the Load Dispatcher. At 1115 hours, on August 4, the unit increased load to 430 MWe, and then at 1510 hours, the unit increased load to 480 MWe, both as requested by the Load Dispatcher. At 0145 hours, on August 5, the unit dropped load to 300 MWe and maintained this load from 0300 hours to 0500 hours, and then increased load to 480 MWe. Open cycle cooling was commenced at 1240 hours and continued until 0030 hours on August 6. During this time, at 1915 hours, on August 5, the unit dropped load to 400 MWe to take the 2A Circulating Water pump out of service due to high differential pressure across the Traveling Screen. The Traveling Screen was cleaned, the 2A Circulating Water pump was returned to service, and at 1955 hours the unit increased load to 490 MWe.

August 10-16: At 0250 hours, on August 10, the unit dropped load to 300 MWe as requested by the Load Dispatcher, and at 0445 hours, the unit increased load to 440 MWe. At 0115 hours, on August 12, the unit dropped load to 285 MWe as requested by the Load Dispatcher, and at 0505 hours, the unit increased load to 440 MWe. At 0000 hours, the unit dropped load to 270 MWe as requested by the Load Dispatcher. At 1040 hours, on August 14, the unit increased load to 500 MWe. At 1510 hours, on August 16, the unit dropped load 50 MWe/hour from 450 MWe due to high Suppression Chamber temperature. This load drop was terminated at 1610 hours, and then the unit increased load to 450 MWe.

August 17-24: Unit Two maintained full load throughout this period. Open cycle cooling was utilized twice, from 1035 hours on August 19 to 0420 hours on August 20, and from 1135 hours to 1755 hours on August 22.

Unit Two (Continued)

August 25-31: At 1100 hours, on August 25, the unit dropped load to 325 MWe to offset the Unit One load increase and maintain a constant Station load. At 0915 hours, on August 29, the unit increased load to 455 MWe. The unit operated on open cycle cooling from 1020 hours to 2200 hours. At 2100 hours, the unit dropped load to 350 MWe as requested by the Load Dispatcher and maintained this load during the remainder of the month.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELA[illegible] AINTENANCE

A. Amendments to Facility License or Technical Specification

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Unit One and Unit Two during the reporting period. This summary includes the following headings: Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q26302	83-21/01T	Reactor Vessel Constant Head Vent	A pinhole leak possibly caused by slag entrapment within the weld.	The leak was not large enough to affect plant operation. Drywell leakage is monitored.	During a short outage, the weld was ground out, NDE inspected, re-welded, and NDE inspected again.
Q26934	83-26/03L	Unit 1 Steam-line Flow Gauge 1-261-2J	The gauge was not zeroed, as a result of instrument drift.	The other gauges were accurate.	The gauge was re-calibrated and zeroed.

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q26209		Suppression Chamber to RHR Pump Circuit Breaker 2-1001-7C	Faulty breaker.	The breaker tripped when the valve reached full closed. The breaker was reset & the valve was operated successfully.	The breaker was replaced.
Q27398	83-11/03L	Suppression Chamber Cooling & Test 2-1001-36B (Valve)	Too high of a setting on the close torque switch.	The valve tripped when the valve was given an open signal. When the operator started the valve manually, it worked properly.	The breaker and overload were replaced; all cubicle connections were checked and the torque switch reset.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

UNIT ONE

Licensee Event Report Number	Date	Title of Occurrence

There were no Licensee Event Reports for Unit One for the reporting period.

UNIT TWO

Licensee Event Report Number	Date	Title of Occurrence
83-12/03L	8-16-83	Torus Temperature Greater Than 95°F

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE 09-08-83

COMPLETED BY Alex Misak

TELEPHONE 309-654-2241x194

OPERATING STATUS

1. Reporting period: 0000 080183 / 2400 083183 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	744.0	5623.6	80794.8
6. Reactor reserve shutdown hours	0.0	0.0	3421.9
7. Hours generator on line	692.0	5531.5	77618.2
8. Unit reserve shutdown hours.	0.0	0.0	909.2
9. Gross thermal energy generated(MWH)	1167091	12630033	158843024
10. Gross electrical energy generated(MWH)	362493	4091609	51213490
11. Net electrical energy generated(MWH)	337974	3848481	47677389
12. Reactor service factor	100.0	96.4	81.5
13. Reactor availability factor	100.0	96.4	85.0
14. Unit service factor	93.0	94.9	78.3
15. Unit availability factor	93.0	94.9	79.2
16. Unit capacity factor (Using MDC)	58.8	85.8	62.5
17. Unit capacity factor (Using Des.MWe)	57.3	83.6	61.0
18. Unit forced outage rate	7.0	2.2	6.4

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

$UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE 09-08-83

COMPLETED BY Alex Misak

TELEPHONE 309-654-2241x194

OPERATING STATUS

0000 080183

1. Reporting period: 2400 083183 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	744.0	5567.0	77830.4
6. Reactor reserve shutdown hours	0.0	0.0	2985.8
7. Hours generator on line	744.0	5537.7	75125.8
8. Unit reserve shutdown hours.	0.0	0.0	702.9
9. Gross thermal energy generated(MWH)	1013860	10692006	155283500
10. Gross electrical energy generated(MWH)	305481	3369805	49407340
11. Net electrical energy generated(MWH)	276649	3138655	46322222
12. Reactor service factor	100.0	95.5	79.3
13. Reactor availability factor	100.0	95.5	82.3
14. Unit service factor	100.0	95.0	76.5
15. Unit availability factor	100.0	95.0	77.2
16. Unit capacity factor (Using MDC)	48.7	70.0	61.3
17. Unit capacity factor (Using Des.MWe)	47.4	68.3	59.8
18. Unit forced outage rate	0.0	1.9	8.6

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup 12-18-83

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

$UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254
UNIT ONE
DATE 09-08-83
COMPLETED BY Alex Misak
TELEPHONE 309-654-2241x194

MONTH August 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	596.8	17.	266.3
2.	697.1	18.	246.6
3.	714.0	19.	479.1
4.	567.8	20.	545.0
5.	425.0	21.	383.0
6.	416.2	22.	494.4
7.	409.3	23.	453.9
8.	414.7	24.	476.1
9.	410.8	25.	515.8
10.	430.4	26.	549.2
11.	524.7	27.	559.4
12.	633.5	28.	548.0
13.	375.9	29.	634.8
14.	159.8	30.	548.1
15.	-17.4	31.	505.4
16.	45.0		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE 09-08-83

COMPLETED BY Alex Misak

TELEPHONE 309-654-2241x194

MONTH August 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	357.6	17.	403.4
2.	378.3	18.	394.3
3.	272.1	19.	405.2
4.	366.3	20.	395.3
5.	414.4	21.	393.7
6.	423.5	22.	396.4
7.	409.0	23.	391.4
8.	421.3	24.	389.5
9.	413.2	25.	339.8
10.	401.4	26.	317.6
11.	414.3	27.	315.6
12.	389.9	28.	316.3
13.	263.8	29.	372.9
14.	376.5	30.	319.4
15.	413.0	31.	329.5
16.	403.7		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit One

COMPLETED BY Alex Misak

DATE September 2, 1983

REPORT MONTH AUGUST 1983

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
83-51	830804	S	0.0	H	5		WA	ZZZZZZ	Reduced load due to Spray Canal temperature
83-52	830813	F	0.0	A	5		HC	HTEXCH	Reduced load due to Main Condenser Vacuum leak
83-53	830814	F	52.0	B	1		HC	HTEXCH	Unit placed in Hot Standby to locate and repair Main Condenser Vacuum leak
83-54	830818	S	0.0	H	5		MB	RECOMB	Reduced load to put Off-Gas Recombiner on line
83-55	830820	S	0.0	H	5		WA	ZZZZZZ	Reduced load due to high Spray Canal temperature
83-56	830822	S	0.0	H	5		WA	ZZZZZZ	Reduced load due to high Spray Canal temperature
83-57	830829	S	0.0	H	5		WA	ZZZZZZ	Reduced load due to high Spray Canal temperature
83-58	830831	S	0.0	B	5		CD	VALVEX	Reduced load to perform Main Steam Isolation Valve Timing

APPROVED

AUG 16 1982

QCOSR

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit Two

COMPLETED BY Alex Misak

DATE September 2, 1983

REPORT MONTH AUGUST 1983

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
83-56	830803	S	0.0	H	5		WA	ZZZZZZ	Reduced load due to high Spray Canal temperature
83-57	830805	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
83-58	830805	F	0.0	B	5		HF	FILTER	Reduced load due to high differential pressure across 2C Circulating Water Pump Traveling Screen
83-59	830810	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
83-60	830812	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
83-61	830813	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
83-62	830816	S	0.0	D	5		SA	VESSEL	Reduced load to cool Suppression Chamber due to high Spray Canal temperature

APPROVED

AUG 16 1982

QCOSR

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit Two

COMPLETED BY Alex Misak

DATE September 2, 1983

REPORT MONTH AUGUST 1983

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
83-63	830825	S	0.0	F	5		EA	ZZZZZZ	Reduced load to offset Unit One increase as requested by Load Dispatcher
83-64	830829	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand

APPROVED
AUG 16 1982
QCOSR

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. MAIN STEAM RELIEF VALVE OPERATIONS

There were no Main Steam Relief Valve Operations for the reporting period.

B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

There was no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: Q1 Reload: 6 Cycle: 7

2. Scheduled date for next refueling shutdown: 9-6-82

3. Scheduled date for restart following refueling: 12-18-82

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: Yes

5. Scheduled date(s) for submitting proposed licensing action and supporting information: 8-19-82: Tech. Spec. changes submitted to the NRC.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
   a) All 7x7 fuel assemblies will be removed from the core.
   b) MAPLHGR curves for fuel types in the core are being extended to 40,000 MWD/ST.
   c) MCPR limits will be determined by GE's ODYN computer code.
   d) The vessel pressure safety limit is being modified to accommodate the potential for higher reactor pressures as calculated by ODYN.

7. The number of fuel assemblies.
   a. Number of assemblies in core: 724
   b. Number of assemblies in spent fuel pool: 800

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
   a. Licensed storage capacity for spent fuel: 3657
   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

APPROVED
APR 20 1978
Q.C.O.S.R.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: Q2 Reload: 6 Cycle: 7

2. Scheduled date for next refueling shutdown: 9-5-83

3. Scheduled date for restart following refueling: 11-12-83

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   No, however, a change to the Technical Specifications is being submitted (see below).

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   June 14, 1983 (Scheduled)

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   a) All new fuel assemblies will be of barrier design; MAPLHGR curves will be re-labeled to include the barrier designation.

   b) The use of improved assumptions in the load reject without bypass analysis resulted in a much improved MCPR operating limit. Technical Specifications are being changed to provide this additional operating margin.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 204

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

APPROVED

APR 20 1978

Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center

Commonwealth Edison
Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

NJK-83-304

August 31, 1983

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555
Attention: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of August 1983.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

bb

Enclosure

IE24
1/1