

## LICENSEE EVENT REPORT

CONTROL BLOCK:                      ①

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	I	A	D	A	C	1	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4	5					
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33					
LICENSEE CODE														LICENSE NUMBER										LICENSE TYPE						CAT 56	

CONT

01	L	6	0	5	0	0	0	3	3	1	7	0	8	0	9	8	3	5	0	9	0	7	8	3	9
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
REPORT SOURCE		DOCKET NUMBER										EVENT DATE						REPORT DATE							

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

02 During normal operation, while conducting surveillance testing, an auxil  
03 iary relay in the main steam isolation logic failed to deenergize. This  
04 relay provides the main steam line high flow signal for one of four chan  
05 nels. The remaining channels functioned properly. LCO entered IAW T.S. T  
06 able 3.2.A. Since this logic is one out of two twice, it would have func  
07 tioned properly had a high flow condition existed. No effect on health a  
08 nd safety of public. No previous similar reportable occurrences.

09	C	D	11	E	12	A	13	R	E	L	A	Y	X	14	A	15	Z	16	8	3	21	0	2	6	27	0	3	L	31	0	32	G	0	8	0	26							
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47			
SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE						COMP. SUBCODE		VALVE SUBCODE		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.		ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NRC-4 FORM 300		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ⑳

10 The relay coil varnish melted causing the relay to fail. The channel wa  
11 s placed in a tripped condition and the relay was replaced. The relay i  
12 s a GE HFA Relay. HFA's have had generic problems of this type. A progr  
13 am has been established to replace all HFA relays with improved models  
14 by the end of the next refueling outage.

15	E	28	0	9	8	29	NA	30	A	31	Surveillance Testing	32	33	34	NA	35	NA	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50											
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION										ACTIVITY CONTENT		AMOUNT OF ACTIVITY		LOCATION OF RELEASE		PERSONNEL EXPOSURES		PERSONNEL INJURIES		LOSS OF OR DAMAGE TO FACILITY		PUBLICATION		ISSUED DESCRIPTION		NAME OF PREPARER		PHONE							

NAME OF PREPARER Daniel R. Kibler

PHONE: 319-851-7306

8309150414 830907  
PDR ADOCK 05000331  
S PDR

NRC USE ONLY

GPO 81-7-225

Iowa Electric Light and Power Company

September 7, 1983

DAEC-83-715

Mr. James G. Keppler  
Regional Administrator  
Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

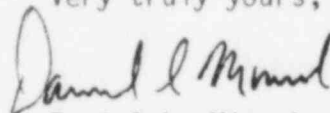
Subject: Licensee Event Report No. 83-026  
(30 day)

File: A-118a, TE-2

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications, Section 6.11.2.b(2), and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report.

Very truly yours,



Daniel L. Mineck  
Plant Superintendent - Nuclear  
Duane Arnold Energy Center

DLM/DRK/pf

Docket 50-331

attachment

cc: Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

NRC Resident Inspector - DAEC

SEP 9 1983

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