

OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE 07/13/83
 COMPLETED BY G. G. Whitney
 TELEPHONE 617-746-7900

OPERATING STATUS

1. Unit Name: Pilgrim 1
 2. Reporting Period: June, 1983
 3. Licensed Thermal Power (MWt): 1998.
 4. Nameplate Rating (Gross MWe): 678.
 5. Design Electrical Rating (Net MWe): 655.
 6. Maximum Dependable Capacity (Gross MWe): 690.
 7. Maximum Dependable Capacity (Net MWe): 670.

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe): None

10. Reasons For Restrictions, If Any:

N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	4343.0	92543.0
12. Number Of Hours Reactor Was Critical	528.2	4096.5	66041.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	509.1	4051.2	63930.4
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	972120.0	7834584.0	110548176.0
17. Gross Electrical Energy Generated (MWH)	332350.0	2717990.0	37044814.0
18. Net Electrical Energy Generated (MWH)	319446.0	2615220.0	35596731.0
19. Unit Service Factor	70.7	93.3	69.1
20. Unit Availability Factor	70.7	93.3	69.1
21. Unit Capacity Factor (Using MDC Net)	66.2	89.9	57.4
22. Unit Capacity Factor (Using DER Net)	67.7	91.9	58.7
23. Unit Forced Outage Rate	17.6	4.5	9.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refuel Outage to commence January, 1984

25. If Shut Down At End Of Report Period, Estimated Date of Startup: Startup commenced on July 2, 1983

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

8307190040 830713
 PDR ADDCK 03000293
 R PDR

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
 UNIT Pilgrim 1
 DATE 07/ 13/83
 COMPLETED BY G. G. Whitney
 TELEPHONE 617-746-7900

MONTH JUNE, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>660.</u>	17	<u>607.</u>
2	<u>661.</u>	18	<u>509.</u>
3	<u>659.</u>	19	<u>617.</u>
4	<u>598.</u>	20	<u>612.</u>
5	<u>662.</u>	21	<u>607.</u>
6	<u>663.</u>	22	<u>599.</u>
7	<u>661.</u>	23	<u>598.</u>
8	<u>660.</u>	24	<u>589.</u>
9	<u>659.</u>	25	<u>547.</u>
10	<u>658.</u>	26	<u>653.</u>
11	<u>29.</u>	27	<u>419.</u>
12	<u>0.</u>	28	<u>0.</u>
13	<u>0.</u>	29	<u>0.</u>
14	<u>0.</u>	30	<u>0.</u>
15	<u>324.</u>	31	<u>—</u>
16	<u>61.</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JUNE, 1983

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 DATE 07/13/83
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 TELEPHONE 617-746-7900

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
8	83/06/11	S	102.2	B	2	N/A	HH	HTEXCH	Repair feedwater heaters and investigate and repair drywell leakage
9	83/06/16	F	18.5	A	3	N/A	HA	INSTRU	Erratic turbine controls
10	83/06/27	F	90.2	B	2	N/A	CD	VALVEX	MSIV Stem packing found leaking in drywell. Repack stem.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next Refueling Shutdown: January, 1984
3. Scheduled date for restart following refueling: April, 1984
- 4.
5. Due to their similarity, requests 4, 5, & 6 are responded to collectively:
6. The fuel, which had been loaded during the 1981 scheduled refueling outage, is of the same P8x8R design, as loaded the previous outage consisting of 112 P8DRB282 assemblies and 60 P8DRB265 assemblies.
7. (a) There are 580 fuel assemblies in the core.
(b) There are 936 fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual spent fuel storage capacity is 1770 fuel assemblies at present.
(b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 834 fuel assemblies.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION

Summary of Operations for June, 1983

The unit was operating at 100% power at the start of the month.

On June 1, the "B" emergency diesel generator was declared inoperable because of a loss of speed control. (Ref: LER 83-33/03L-0). A power reduction was made on June 4 to backwash the condensers, returning to full load the same day.

On June 5, the "C" SSW pump was made inoperable to correct vibration problems. and perform preventive maintenance. On June 6, the AO 5033B nitrogen purge valve was made inoperable when it could not pass the required timing surveillance test (Ref: LER 83-35/03L-0). The "B" D/G was returned to service the same day.

On June 10, the RWCU System was removed from service in an attempt to identify the source of increasing drywell leakage. A scheduled feedwater heater maintenance outage and drywell entry was started on June 11. Drywell inspection found a leaking hinge pin cover on a feedwater check valve. The 1400-4A MOV core spray test valve was found with 3 out of 4 operator mounting cap screws missing (Ref: LER 83-34/03L-0). Various steam leaks were also repaired in the condenser area. Anodes were changed in the water boxes and a ground was repaired on the "B" 125 VDC battery system. Two drywell air flow dampers were found malfunctioning ; an air flow balance was performed after their repair. A seismic hanger was found with loose bolts on the "A" RHR loop. The bolts were tightened and the reactor made critical on June 14 at 2000 hrs. and the unit placed on line at 0808 hrs. on June 15. On June 16, while reducing power for a rod pattern change the unit tripped at 0030 hrs. because of erratic EPR control. The "C" S/RV was used for pressure control during this scram. After adjustment, the reactor was made critical at 1157 hrs, the unit placed on line at 1900 hrs. and 100% power was achieved on June 17. Later the same day, PNPS was notified by BECo Engineering Dept. that the "A" spent fuel pool heat exchanger support did not meet the required seismic loading criteria. (Ref: LER 83-29/01T-0). The "A" & "B" SFP loops were isolated with the "B" loop used only as required until the support was returned to original design on June 25.

On June 18, power was reduced for a condenser backwash and control rod exercises. Control rod 10-31 could not be withdrawn and the symmetric rods were inserted. The maximum power available with this pattern was approximately 96%. The "C" SSW pump was returned to service on this day.

On June 25, power was reduced for a condenser backwash and returned to full available power the same day.

On June 27, the unit was taken off-line because of increasing drywell leakage. Drywell entry and investigation found MO 202-4B and 202 5A recirculation valves leaking. MSIV "B" stem packing was also found to be leaking. Repairs to the "B" 5th point heater, condenser tube cleaning and seal weld leaks in the drywell were also made during this outage. The unit remained off-line until the end of the month.

NOTES:

1. The condensers were backwashed every weekend because of horseshoe crab intrusion.
 2. The annual emergency preparedness drill was held on June 29.
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SAFETY/RELIEF VALVE CHALLENGES

MONTH OF JUNE, 1983

Requirement: T.A.P. II.K.3.3
Date: June 16, 1983
Valve #: 203-3C
Reactor Pressure: 980 psi
Reason: Manual actuation of the valve to control reactor pressure following a scram which was caused by MSIV initiation.

PILGRIM NUCLEAR POWER STATION

Month JUNE, 1983

MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
CAC	AO 5033B	Failed time test	Oil in vlv.	Replace solenoid	PDCR to replace solenoid with another model.	83-035/03L-0
RWCU	MO 1201-2	Packing leak	Packing	Inspected MO and replaced valve	None-routine maintenance	N/A
SSW	'C' SSW pump	Vibration	Rebuilt pump & motor	Balance	None-routine maintenance	N/A
I&C	IRM 'A' switch	Would not select	misalign- ment	Readjusted switch	Will overhaul during refuel outage	N/A
Elec. pwr system	'B' 125 VDC	Ground	Battery tracking	Cleaned battery	Establish schedule to clean periodically	N/A
Elec. pwr system	'A' Battery	Ground	Steam leak	Replaced MSIV 'B' Inboard limit switch & repacked valve	None-routine maintenance (and installed plug in junction box)	N/A
Standby Gas	AO 95	No indication	Limit switch	Readjusted limit switch	Inspected every three months	N/A
D/G Aux.	'B' D/G	Would not take full load	Governor	Replaced governor	Not servicable at site-will ship to vendor for overhaul.	83-033/03L-0
Core Spray	MOV 1400-4A	Mounting bolts missing	Vibration	Replace bolts	Applied thread locking compound and torqued to a higher Ft/lbs.	83-034/03L-0
RBCCW	Spent fuel pool Hx.E206-A	Initial construction error	Unknown	Return to original design	None - isolated incident	83-029/01T-0

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

July 13, 1983

BECO Ltr. #83-186

Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket No. 50-293
License DPR-35

Subject: June, 1983 Monthly Report

Dear Sir:

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning.

Respectfully submitted,

W D Harrington
William D. Harrington

:em

cc: Regional Administrator, Region 1
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Standard BECo Monthly Report Distribution

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