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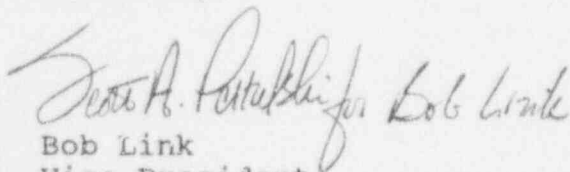
Gentlemen:

DOCKET 50-301
LICENSEE EVENT REPORT 94-003-00
STEAM GENERATOR TUBE DEGRADATION
POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report (LER) 94-003-00 for Point Beach Nuclear Plant (PBNP), Unit 2. This report is provided in accordance with PBNP Technical Specification 15.4.2.A.7(c), "Reports required by Table 15.4.2-1, 'Steam Generator Tube Inspection,' shall provide the information required by Technical Specification 15.4.2.A.7(b) and a description of investigations conducted to determine the cause of the tube degradation and corrective actions taken to prevent recurrence. The report shall be submitted to the Commission prior to resumption of plant operation." LER 94-003-00 is filed in accordance with Technical Specification Table 15.4.2-1 under the reporting requirement of 10 CFR 50.73(a)(2)(ii), "The licensee shall report...any event or condition that resulted in the condition of the nuclear power plant, including its principal safety barriers, being seriously degraded..."

Please contact us if any further information is required.

Sincerely,


Bob Link
Vice President
Nuclear Power

DAW/jg
Enclosure

cc: NRC Resident Inspector
NRC Regional Administrator

9411020185 941026
PDR ADOCK 05000301
S PDR

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LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Point Beach Nuclear Plant, Unit 2

DOCKET NUMBER (2)

05000301

PAGE (3)

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TITLE (4)

Steam Generator Tube Degradation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	10	94	94	-- 003 --	00	10	26	94	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)		000	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)	
			20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)	
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER	
			20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)	
			20.405(a)(1)(iv)		X 50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)			
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)

NAME

David A. Weaver, Senior Engineer - Licensing

TELEPHONE NUMBER (Include Area Code)

(414) 221-3418

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	AB	SG	W120	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES

(If yes, complete EXPECTED SUBMISSION DATE).

X

NO

EXPECTED SUBMISSION DATE (15)

MONTH

DAY

YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

Point Beach Nuclear Plant (PBNP) Unit 2 was shut down for Refueling 20 (U2R20) on September 24, 1994. Leak testing and eddy current examination of the steam generator tubes began on October 5, 1994, and was completed on October 12, 1994. Eddy current testing of the "A" steam generator revealed 4 tubes degraded $\geq 40\%$ of the wall thickness, 8 tubes with axial indications in the tube-end area, and 155 sleeved tubes with parent tube circumferential indications. All of these tubes (167 total) were plugged.

In the "B" steam generator, eddy current testing revealed 1 tube degraded $\geq 40\%$ of the wall thickness, 10 tubes with axial indications in the tube-end area, and 67 sleeved tubes with parent tube circumferential indications. All of these tubes (78 total) were plugged.

The 800 psid secondary-to-primary leak test revealed 6 sleeves and 2 open tubes with dampness in the "A" steam generator and 13 open tubes with dampness in the "B" steam generator. These tubes remain in service. The 800 psid leak test also revealed 2 wet mechanical plugs in the "A" steam generator and 30 wet mechanical plugs in the "B" steam generator. All of these plugs (32 total) were replaced with new mechanical plugs.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT DESCRIPTION800 PSID Leak Test

Prior to eddy current inspection on both steam generators, 800 psid secondary-to-primary leak tests were performed in each steam generator. Remote video equipment was utilized to inspect for leakage at the primary tubesheet face. The results of the leak test follow:

Extent of Inspection	Steam Generator	
	"A"	"B"
Tubes with evidence of potential leakage (<3 drops/min)	2	13
Mechanical plugs with evidence of potential leakage (<=4 drops/min)	2	31
Explosive plugs with excessive leakage (10-12 drops/min)	0	0
Sleeved tubes with evidence of potential leakage (<=4 drops/min)	6	0
Total	10	44

Most of the moisture observed during the tests was believed to be either condensation from the tubesheet, trapped water in the recesses of plugs and sleeves, or minor seepage from sleeves. Based on the slightly elevated primary-to-secondary leakage in the Unit 2 steam generators during the last operating cycle (approximately 40 gal/day leakage), all leaking plugs were removed and replaced.

Eddy Current Testing

Eddy current testing began shortly after the leak tests were completed. The eddy current program utilized the CECCO-5 probe and included the following scope:

1. A 100% full-length inspection of all in-service unsleeved tubes. (PBNP Technical Specifications require a 3% sample including all previously degraded tubes that had not been repaired). Prior to this outage, 366 of 3260 tubes in the "A" steam generator and 334 of 3260 tubes in the "B" steam generator had been plugged.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

2. Previously degraded tubes that had not been repaired.
3. All hot leg distorted indications were reexamined using the rotating pancake coil (RPC) inspection method to detect axial tube degradation.
4. All hot leg and a 20% sample of cold leg sleeves were inspected for circumferential indications.

"A" Steam Generator Tube Plugging

On October 12, 1994, review and verification of all eddy current data for tubes with indications exceeding the plugging limit were completed. A total of 167 tubes were plugged in the "A" steam generator. Four tubes were found having degradation \geq the plugging limit of 40% of the nominal wall thickness (PBNP Technical Specification 15.4.2.A.5). However, 8 tubes with axial indications were plugged as a preventive measure. In addition, 155 sleeved tubes were plugged due to parent tube circumferential indications in the upper hybrid expansion joint (HEJ), and 2 tubes were re-plugged after removing leaking hot leg plugs.

Below is a list of affected tubes in the "A" steam generator, including a list of abbreviations used in the following two tables:

HL - Hot Leg	CL - Cold Leg
TEH - Tube end (HL)	USJ - Top of Sleeve Indication (HL)
SAI - Single Axial Indication	2H - #2 Tube Support Plate (HL)
MAI - Multiple Axial Indications	6H - #6 Tube Support Plate (HL)
TRH - Tube end Roll (HL)	1C - #1 Tube Support Plate (CL)
RST - Restricted Tube	LPI - Lower Sleeve Indication
UPI - Upper Sleeve Indication	1H - #1 Tube Support Plate (HL)
TSC - Tubesheet (CL)	

Tubes plugged in "A" Steam Generator		
Tube	Defect	Location
R04C06	MAI	TEH 2.2
R18C06	47%	1H 0.1
R13C18	LPI	USJ 0.8
R16C18	LPI	USJ 0.8

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REDUCTION PROJECT (3150-0104), OFFICE OF
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Tubes plugged in "A" Steam Generator		
Tube	Defect	Location
R19C18	LPI	USJ 0.3
R17C18	LPI	USJ 0.2
R20C19	LPI	USJ 0.0
R16C19	LPI	USJ 0.4
R17C19	LPI	USJ 0.3
R36C19	MAI	TEH 2.6
R33C20	MAI	TEH 2.3
R04C20	LPI	USJ 0.0
R22C20	LPI	USJ 0.3
R23C21	LPI	USJ 0.2
R06C21	LPI	USJ 0.4
R17C21	LPI	USJ 0.0
R16C21	LPI	USJ 0.0
R04C21	LPI	USJ 0.1
R20C21	LPI	USJ 0.4
R14C21	LPI	USJ 0.0
R13C22	LPI	USJ 0.4
R03C22	LPI	USJ 0.4
R16C22	LPI	USJ 0.7
R14C22	LPI	USJ 0.2
R25C23	LPI	USJ 0.3
R24C23	LPI	USJ 0.2
R03C23	LPI	USJ 0.2
R02C23	LPI	USJ 0.3

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Tubes plugged in "A" Steam Generator		
Tube	Defect	Location
R04C24	LPI	USJ 0.7
R03C24	LPI	USJ 0.3
R04C25	LPI	USJ 0.4
R19C25	LPI	USJ 0.4
R26C25	LPI	USJ 0.3
R26C26	LPI	USJ 0.4
R05C27	LPI	USJ 0.3
R15C27	LPI	USJ 0.4
R14C28	LPI	USJ 0.3
R06C28	LPI	USJ 0.2
R07C31	LPI	USJ 0.4
R28C31	LPI	USJ 0.3
R09C32	LPI	USJ 0.5
R25C32	UPI	USJ 2.0
R18C32	UPI	USJ 3.3
R14C32	LPI	USJ 0.2
R10C33	LPI	USJ 0.3
R11C33	LPI	USJ 0.5
R08C33	UPI	USJ 2.6
R18C33	LPI	USJ 0.6
R04C34	UPI	USJ 2.6
R05C34	LPI	USJ 0.5
R09C34	LPI	USJ 0.6
R28C34	LPI	USJ 0.0

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Tubes plugged in "A" Steam Generator		
Tube	Defect	Location
R04C35	LPI	USJ 0.6
R05C35	LPI	USJ 0.3
R29C35	LPI	USJ 0.0
R09C35	LPI	USJ 0.4
R18C36	UPI	USJ 2.6
R23C36	LPI	USJ 0.2
R10C37	UPI	USJ 2.6
R41C37	MAI	TEH 6.4
R31C37	LPI	USJ 0.4
R19C37	UPI	USJ 2.5
R07C37	LPI	USJ 0.1
R15C37	LPI	USJ 0.3
R29C37	LPI	USJ 0.0
R23C38	LPI	USJ 0.6
R09C38*	UPI	USJ 2.4
R06C39	LPI	USJ 0.2
R17C39	LPI	USJ 0.3
R07C39	UPI	USJ 2.9
R09C39	UPI	USJ 2.6
R12C39	LPI	USJ 0.5
R06C40	LPI	USJ 0.5
R05C40	UPI	USJ 3.0
R16C40	LPI	USJ 0.4
R31C41	LPI	USJ 0.6

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WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK
REDUCTION PROJECT (3150-0104), OFFICE OF
MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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Tubes plugged in "A" Steam Generator		
Tube	Defect	Location
R19C41	UPI	USJ 2.9
R05C41	UPI	USJ 2.6
R06C42	LPI	USJ 0.3
R15C42	LPI	USJ 0.3
R17C42	LPI	USJ 0.5
R10C42	LPI	USJ 0.6
R29C42	LPI	USJ -0.1
R31C42	LPI	USJ 0.3
R09C43	LPI	USJ 0.4
R04C43	LPI	USJ 0.3
R30C43	LPI	USJ 0.0
R07C43	LPI	USJ 0.2
R10C43	LPI	USJ 0.4
R30C44	LPI	USJ 0.5
R31C45	LPI	USJ 0.2
R16C45	LPI	USJ 0.2
R06C46	UPI	USJ 2.9
R09C47	UPI	USJ 2.7
R04C47	LPI	USJ 0.3
R19C47	LPI	USJ 0.4
R08C47	LPI	USJ 0.5
R32C47	UPI	USJ 2.8
R17C47	LPI	USJ 0.3
R07C49	LPI	USJ 1.3

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Tubes plugged in "A" Steam Generator		
Tube	Defect	Location
R32C49	LPI	USJ 0.2
R17C50	LPI	USJ 0.3
R08C51	UPI	USJ 2.4
R09C51	UPI	USJ 2.7
R14C51	UPI	USJ 2.6
R19C52	LPI	USJ 0.4
R29C53	LPI	USJ 0.3
R08C54	LPI	USJ 0.1
R06C54	UPI	USJ 2.3
R05C55	LPI	USJ 0.3
R20C55	UPI	USJ 4.2
R17C55	LPI	USJ 0.2
R30C55	UPI	USJ 4.5
R15C56	LPI	USJ 0.2
R26C56	LPI	USJ 0.3
R22C56	UPI	USJ 1.8
R14C56	LPI	USJ 0.0
R16C57	UPI	USJ 2.7
R25C57	LPI	USJ 0.0
R34C57	SAI	TEH 2.4
R20C57	UPI	USJ 3.6
R14C57	LPI	USJ 0.2
R09C58	41%	TSC 0.5
R06C58	LPI	USJ 0.4

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Tubes plugged in "A" Steam Generator		
Tube	Defect	Location
R04C59	UPI	USJ 4.4
R02C59	UPI	USJ 3.2
R09C59	LPI	USJ 0.2
R06C60	UPI	USJ 3.3
R04C60	UPI	USJ 2.7
R36C60	MAI	TEH 2.9
R07C60	LPI	USJ 0.3
R40C62	87%	TEH 7.4
R03C62	LPI	USJ 0.4
R09C63	LPI	USJ 0.4
R25C63	LPI	USJ 0.6
R17C63	LPI	USJ 0.3
R23C65	LPI	USJ 0.2
R02C65	UPI	USJ 3.0
R27C65	LPI	USJ 0.2
R13C66	LPI	USJ 0.3
R09C66	UPI	USJ 3.6
R21C66	LPI	USJ 0.6
R03C66	UPI	USJ 3.5
R14C66	LPI	USJ 0.2
R05C67	LPI	USJ 0.3
R03C67	LPI	USJ 0.0
R25C67	LPI	USJ 0.3
R24C67	LPI	USJ 0.3

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TEXT (if more space is required, use additional copies of NRC Form 366A) (17)

Tubes plugged in "A" Steam Generator		
Tube	Defect	Location
R22C67	LPI	USJ 0.3
R14C67	LPI	USJ 0.1
R24C68	LPI	USJ 0.3
R13C68	LPI	USJ 0.5
R04C68	LPI	USJ 0.0
R13C69	UPI	USJ 3.5
R12C70	LPI	USJ 0.3
R18C70	LPI	USJ 0.5
R37C70	MAI	TEH 3.0
R14C72	LPI	USJ 0.3
R21C73	LPI	USJ 0.6
R19C73	UPI	USJ 4.2
R12C75	LPI	USJ 0.5
R11C75	LPI	USJ 0.3
R08C76	LPI	USJ 0.5
R14C76	LPI	USJ 0.3
R16C76	LPI	USJ 0.4
R05C77	80%	TEH 12.1
R16C87	MAI	TEH 2.4

* Weld plug due to tube end concerns.

"B" Steam Generator Tube Plugging

In the "B" steam generator, a total of 78 tubes were plugged. One tube was found with degradation \geq the plugging limit of 40% of the nominal wall thickness. In addition, 11 tubes with axial indications in the

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

tube-end area and one restricted tube were plugged as a preventive measure. 66 sleeved tubes were plugged as a result of parent tube circumferential indications in the upper sleeve joint.

Tubes plugged in "B" Steam Generator		
Tube	Defect	Location
R23C12	SAI	TEH 8.1
R26C16	SAI	TEH 9.9
R25C16	SAI	TEH 6.2
R10C18	UPI	USJ 3.4
R19C22	UPI	USJ 2.5
R12C23	UPI	USJ 3.2
R14C23	UPI	USJ 2.4
R03C23	UPI	USJ 2.3
R19C24	UPI	USJ 2.4
R08C24	UPI	USJ 2.7
R16C24	UPI	USJ 2.7
R04C26	UPI	USJ 2.5
R04C27	LPI	USJ 0.3
R03C28	UPI	USJ 3.4
R11C29	UPI	USJ 3.5
R25C31	UPI	USJ 2.8
R39C31	69%	TEH 6.1
R14C32	UPI	USJ 0.5
R16C33	UPI	USJ 3.5
R27C33	UPI	USJ 3.4
R10C34	UPI	USJ 2.8
R16C34	UPI	USJ 3.4

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATIONESTIMATED BURDEN PER RESPONSE TO COMPLY WITH
THIS INFORMATION COLLECTION REQUEST: 50.0 HRS.
FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO
THE INFORMATION AND RECORDS MANAGEMENT BRANCH
(MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION,
WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK
REDUCTION PROJECT (3150-0104), OFFICE OF
MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Tubes plugged in "B" Steam Generator		
Tube	Defect	Location
R28C34	UPI	USJ 2.8
R04C35	LPI	USJ 0.0
R06C35	UPI	USJ 3.7
R25C36	UPI	USJ 2.3
R18C36	UPI	USJ 3.0
R18C37	UPI	USJ 2.7
R18C38	UPI	USJ 3.8
R16C38	UPI	USJ 1.2
R18C39	UPI	USJ 3.7
R16C39	UPI	USJ 3.4
R30C40	UPI	USJ 2.7
R18C41	UPI	USJ 3.5
R04C42	LPI	USJ 0.3
R29C43	UPI	USJ 2.4
R17C43	LPI	USJ 0.4
R16C44	LPI	USJ 0.2
R27C44	UPI	USJ 2.4
R32C44	UPI	USJ 2.7
R23C45	LPI	USJ 0.1
R29C45	LPI	USJ 0.3
R24C47	UPI	USJ 3.2
R25C47	UPI	USJ 4.7
R26C47	UPI	USJ 2.8
R23C53	LPI	USJ 0.8

LICENSEE EVENT REPORT (LER)
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TEXT (if more space is required, use additional copies of NRC Form 366A) (17)

Tubes plugged in "B" Steam Generator		
Tube	Defect	Location
R30C53	UPI	USJ 3.2
R10C55	UPI	USJ 2.9
R03C55	UPI	USJ 1.2
R05C56	UPI	USJ 2.3
R21C56	UPI	USJ 3.1
R04C57	UPI	USJ 2.2
R07C61	UPI	USJ 3.9
R03C62	UPI	USJ 2.5
R06C63	UPI	USJ 2.1
R05C63	UPI	USJ 2.6
R03C63	UPI	USJ 2.3
R13C64	LPI	USJ 0.7
R18C64	LPI	USJ 0.2
R03C66	UPI	USJ 1.1
R04C66	UPI	USJ 1.3
R06C66	UPI	USJ 4.5
R33C67	MAI	TEH 7.1
R05C67	UPI	USJ 4.1
R09C67	UPI	USJ 1.5
R04C68	UPI	USJ 2.8
R20C70	UPI	USJ 3.2
R08C71	UPI	USJ 3.0
R10C71	UPI	USJ 3.5
R18C72	UPI	USJ 3.3

LICENSEE EVENT REPORT (LER)
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Tubes plugged in "B" Steam Generator		
Tube	Defect	Location
R08C73	UPI	USJ 3.2
R03C83	SAI	TEH 0.3
R14C83	MAI	TEH 3.5
R05C86	MAI	TEH 3.0
R10C86	SAI	TEH 7.8
R08C86	MAI	TEH 3.9
R14C86	SAI	TEH 7.6
R14C87	MAI	TEH 6.5

STEAM GENERATOR TUBE PLUG REPAIRS

Due to administrative leak control initiatives to reduce at-power leakage, plug repair efforts were necessary during this outage. Two plugs in the "A" steam generator and 30 plugs in the "B" steam generator were replaced. The locations of the replaced plugs are listed below:

Plugs replaced in "A" Steam Generator			
R01C88	R07C36	NA	NA

Plugs replaced in "B" Steam Generator			
R04C59	R09C65	R15C53	R23C57
R05C47	R11C43	R18C40	R24C59
R07C27	R11C46	R18C49	R25C54
R07C45	R11C47	R18C50	R25C55
R07C46	R11C48	R19C68	R32C47
R08C47	R12C41	R21C41	R32C48
R09C46	R12C46	R22C49	NA
R09C53	R13C54	R22C53	NA

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COMPARISON TO LAST YEAR'S RESULTS

Last year's examination was compared to this year's data to determine growth rates and/or trending. The results are as follows:

1. In the "A" steam generator, 167 tubes were plugged compared to 21 tubes in 1993, totalling 533 of 3260 tubes plugged thus far. The "B" steam generator required 78 tubes to be plugged compared to 12 tubes in 1993, totalling 412 of 3260 tubes plugged thus far. However, this is the first time sleeved tube circumferential indication acquisition was attempted at PBNP.
2. A comparison of the data revealed the average growth rate of previous indications to be approximately 5 to 7 percent/year with new indications found during the full-length tests. These indications will be monitored during future examinations as required by Technical Specification 15.4.2.

STEAM GENERATOR TUBESHEET CLEANING

Sludge lancing of the steam generator tubesheet areas removed 74 pounds of sludge from the "A" steam generator (compared to 134 pounds in 1993) and 51 pounds from the "B" steam generator (compared to 198 pounds in 1993). Post cleaning checks were performed on both steam generators to verify the effectiveness of the cleaning.

COLD LEG WASTAGE AND PITTING

A review of the geometry of the indications verified that tube pitting continues not to be a problem at Point Beach Nuclear Plant, Unit 2. Minimal growth rates were encountered at the top of the tubesheet which had been the historical location of cold leg damage. One tube was plugged in the "A" steam generator as a result of >40% throughwall degradation in the cold leg.

TUBESHEET CREVICE CORROSION

18 plugs were installed as a result of tubesheet crevice corrosion in the tube-end area. Tubesheet crevice corrosion is still active in both steam generators and is continually monitored.

PROBE RESTRICTIONS

In the "A" steam generator, 68 tubes would not pass a .720 inch probe; 11 of these tubes would not pass a .700 inch probe. A .680 inch probe

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was able to pass through 7 of these tubes, and a .650 inch diameter probe passed through the remaining 4 tubes.

In the "B" steam generator, 25 tubes would not pass a .720 inch probe; 8 of these tubes would not pass a .700 inch probe. A .680 inch probe was able to pass through 2 of these tubes, and a .650 inch diameter probe passed through the remaining 6 tubes.

STEAM GENERATOR CLOSEOUT INSPECTIONS

Closeout inspections were performed following the maintenance of each steam generator. No abnormalities were encountered during the primary closeout inspection.

REPORTABILITY

This report is provided in accordance with PBNP Technical Specification 15.4.2.A.7(c), "Reports required by Table 15.4.2-1, 'Steam Generator Tube Inspection,' shall provide the information required by Technical Specification 15.4.2.A.7(b) and a description of investigations conducted to determine the cause of the tube degradation and corrective actions taken to prevent recurrence. The report shall be submitted to the Commission prior to resumption of plant operation." Since more than 1% of the inspected steam generator tubes were considered defective, both steam generators were categorized as C-3 in accordance with Technical Specification 15.4.2.A.2(b). LER 94-003-00 is filed in accordance with Technical Specification Table 15.4.2-1 under the reporting requirement of 10 CFR 50.73(a)(2)(ii), "The licensee shall report...any event or condition that resulted in the condition of the nuclear power plant, including its principal safety barriers, being seriously degraded..."