

CONTROL BLOCK

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | C | A | S | 0 | S | 2 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

CONT

01 | L | 0 | 5 | 0 | 0 | 0 | 3 | 6 | 1 | 7 | 0 | 6 | 0 | 2 | 8 | 3 | 8 | 0 | 7 | 0 | 5 | 8 | 3 | 9

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | With Unit 2 in Mode 1 and the Core Operating Limit Supervisory System (COLSS) out

03 | of service, Surveillance Test S023-3-3.6 failed to verify that the DNBR was with-

04 | in acceptable limits, therefore, at 1245 the Action Statement associated with

05 | LCO 3.2.4 was entered. In accordance with the Action Statement steps were taken

06 | within 15 minutes to restore the DNBR to within allowable limits. The Action

07 | Statement was satisfied at 1320 when COLSS was returned to service and the DNBR

08 | power operating limit verified satisfactory. Public health and safety were not

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

09 | I | A | 11 | B | 12 | A | 13 | I | N | S | T | R | U | 14 | Y | 15 | Z | 16

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

17 | 8 | 3 | 0 | 6 | 4 | 0 | 3 | L | 0

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

18 | E | 19 | X | 20 | Z | 21 | Z | 22 | 0 | 0 | 0 | 0 | 23 | Y | 24 | N | 25 | N | 26 | Z | 27 | 9 | 28 | 9 | 29 | 9

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NFRD-4 FORM SUB PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | This event occurred primarily because the DNBR was calculated using an overly

11 | conservative thermal power uncertainty bias term. As corrective action this term

12 | will be changed by incorporating a more accurate core flow measurement, calculated

13 | via the calorimetric rather than the vessel delta-p method. (See Attachment.)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

15 | B | 28 | 0 | 8 | 0 | 29 | NA | 30 | A | 31 | Operator Observation | 32

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

16 | Z | 33 | Z | 34 | NA | 35 | NA | 36

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE

17 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION

18 | 0 | 0 | 0 | 40 | NA | 41

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

PERSONNEL INJURIES NUMBER DESCRIPTION

19 | Z | 42 | 8307180135 830705 PDR ADOCK 05000361 S PDR

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION

20 | N | 44 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

PUBLICITY ISSUED DESCRIPTION

NAME OF PREPARER H. B. RAY

PHONE: 714/492-7700

NRC USE ONLY

ATTACHMENT TO LER 83-064
SOUTHERN CALIFORNIA EDISON COMPANY
SAN ONOFRE NUCLEAR GENERATING STATION
UNIT NO. 2, DOCKET NO. 50-361

SUPPLEMENTAL INFORMATION FOR CAUSE DESCRIPTION AND CORRECTIVE
ACTIONS:

Test data obtained at 100% power testing permits decreased uncertainty in the core flow measurement which will remove most of the bulk uncertainty in the DNBR calculation. This action is considered to be a normal evolution during plant start-up.

A contributing factor was the Channel "D" Center Ex-core Detector which read 2 - 3% higher than the other three center detectors. Investigation revealed that its amplifier card had drifted slightly high and it was recalibrated. This was considered to be an isolated occurrence. No further corrective action is necessary.

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION
P.O. BOX 128
SAN CLEMENTE, CALIFORNIA 92672

H. B. RAY
STATION MANAGER

July 5, 1983

RECEIVED
MRC SCE

1983 JUL 11 AM 11:01

REGION V
TELEPHONE
(714) 492-7700

U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J.B. Martin, Regional Administrator

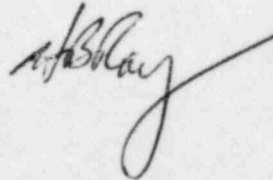
Dear Sir:

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 83-064
San Onofre Nuclear Generating Station, Unit 2

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-10 for San Onofre Unit 2, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for an occurrence involving Limiting Condition for Operation (LCO) 3.2.4 associated with the Departure from Nucleate Boiling Ratio (DNBR) margin. Enclosed LER 83-064 addresses this event, including corrective actions to prevent recurrence.

If there are any questions regarding the above, please contact me.

Sincerely,



Enclosure: LER No. 83-064

1/1 IE-22
83-287

cc: A.E. Chaffee (USNRC Resident Inspector, Units 2 and 3)
J.P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

U.S. Nuclear Regulatory Commission
Office of Management Information and Program Control (MIPC)

Institute of Nuclear Power Operations (INPO)