

U. S. NUCLEAR REGULATORY COMMISSION
UPDATE REPORT
PREVIOUS REPORT DATE 8/4/83

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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REPORT SOURCE L 6 0 5 0 0 0 3 6 6 7 0 7 0 9 8 3 8 0 8 1 8 8 3 9

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 While investigating the first two events of this LER, it was discovered

0 3 that RCIC coupling end bearing temperature switch 2E51-N753 had been

0 4 removed and a temperature indicator of a different model (local)

0 5 installed in its place without first initiating a Design Change Request

0 6 per the "PLANT MODIFICATIONS APPROVAL AND IMPLEMENTATION" procedure

0 7 (HNP-809). This event is contrary to Tech. Specs. section 6.9.1.9.c.

Public health and safety were not affected by this non-repetitive event.

SYSTEM CODE C E		CAUSE CODE A		CAUSE SUBCODE C		COMPONENT CODE I N S T R U				COMP. SUBCODE E		VALVE SUBCODE Z	
11		12		13		14				15		16	
EVENT YEAR 8 3		SEMI-ANNUAL REPORT NO. 0 5 0		OCCURRENCE CODE 0 3		REPORT TYPE X		REVISION NO. 1					
21		22		23		24		25					
ACTION TAKEN X	FUTURE ACTION Z	EFFECT ON PLANT Z	SHUTDOWN METHOD Z	HOURS 0 0 0 0	ATTACHMENT SUBMITTED Y	NPRD-4 FORM SUB. N	PRIME COMP. SUPPLIER N	COMPONENT MANUFACTURER A 5 0 1					
18	19	20	21	22	23	24	25	26					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 This event is the result of personnel error. Responsible personnel have
1 1 been counseled concerning the necessity of complying with administrative
1 2 controls. A Design Change Request is currently being written.
1 3

1	4	8	9											80						
FACILITY STATUS				% POWER				OTHER STATUS				METHOD OF DISCOVERY				DISCOVERY DESCRIPTION				80
1	5	C	28	0	0	0	29	NA	A	31	Event Investigation							32	80	
ACTIVITY CONTENT				RELEASED OF RELEASE				AMOUNT OF ACTIVITY								LOCATION OF RELEASE				80
1	6	Z	33	Z	34	NA				NA				36				80		
PERSONNEL EXPOSURES				DESCRIPTION																80
1	7	0	0	0	37	Z	38	NA								39	80			
PERSONNEL INJURIES				DESCRIPTION																80
1	8	0	0	0	40	NA											41	80		
LOSS OF OR DAMAGE TO FACILITY				DESCRIPTION																80
1	9	Z	42	NA											43	80				
PUBLICITY				ISSUED				DESCRIPTION												80
2	0	N	44	NA				8309070396 830818 PDR ADOCK 05000366 S PDR							NRC USE ONLY				80	
																				80

NAME OF PREPARER

S. B. Tipps

PHONE

(912) 367-7851

NARRATIVE REPORT
FOR LER 50-366/1983-050, Rev. 1
Update Report Previous Report Date 08/04/83

LICENSEE : GEORGIA POWER COMPANY
FACILITY NAME : EDWIN I. HATCH
DOCKET NUMBER : 50-366

Tech. Specs. section(s) which requires report:

1. This 30 day LER is required by Tech. Specs. section 6.9.1.9.b, because this event showed that the unit was not meeting the requirements of Tech. Specs. section 3.7.3, ACTION b.
2. This 30 day LER is required by Tech. Specs. section 6.9.1.9.b, because this event showed that the unit was not meeting the requirements of Tech. Specs. section 3.7.3.
3. This 30 day LER is required by Tech. Specs. section 6.9.1.9.c., because the event showed a lack of administrative controls.

Plant conditions at the time of the event(s):

1. The plant was in startup and hot standby at approximately 165 PSIG reactor pressure (1% power).
2. The plant was in startup and hot standby at approximately 150 PSIG reactor pressure (0% power).
3. The plant was in the run mode at 1503 MWT (approximately 62% power).

Detailed description of the event(s):

1. This event is a failure to prove RCIC operability within 12 hours after steam pressure was adequate to perform the test as required by Tech. Specs. section 3.7.3., ACTION b .
2. On 07/09/83, after performance of the "RCIC PUMP OPERABILITY" procedure (HNP-2-3405), operations personnel were attempting to place RCIC in normal standby configuration. When the operator reset the automatic initiation signal, the steam supply valve (2E51-F045) opened and RCIC started. RCIC was declared inoperable. This event is contrary to the requirements of Tech. Specs. section 3.7.3.

3. On 07/27/83, during investigation of the above events it was discovered that the RCIC coupling end bearing temperature switch (2E51-N753) had been removed and a temperature indicator of a different model (local indicating) was installed in its place without first initiating a Design Change Request as required by the "PLANT MODIFICATIONS APPROVAL AND IMPLEMENTATION" procedure (HNP-809). This event is contrary to the requirements of Tech. Specs. section 6.9.1.9.c.

Consequences of the event(s):

1. This event had no effect on plant operations. The health and safety of the public were not affected by this event.
2. This event had no effect on plant operations. The health and safety of the public were not affected by this event.
3. This event had no effect on plant operations. The health and safety of the public were not affected by this event.

Status of redundant or backup subsystems and/or systems:

1. HPCI was operable during this event.
2. HPCI was operable during this event.
3. HPCI was operable during this event.

Justification for continued operation:

1. Operation was continued as permitted by Tech. Specs. section 3.7.3, ACTION a.
2. Operation was continued as permitted by Tech. Specs. section 3.7.3, ACTION a.
3. A standing order was issued per the "PLANT OPERATING ORDERS" procedure (HNP-12) on 07-09-83, which requires a visual check of bearing temperature after any RCIC start, and every 30 minutes thereafter while it is running.

If repetitive, number of previous LER:

1. This event is non-repetitive.
2. This event is non-repetitive.
3. This event is non-repetitive.

Impact to other systems and/or Unit:

1. This event had no impact upon any other Unit 2 system, nor on Unit 1.
2. This event had no impact upon any other Unit 2 system, nor on Unit 1.
3. This event had no impact upon any other Unit 2 system, nor on Unit 1.

Cause(s) of the event(s):

1. While personnel were attempting to prove RCIC operability, a damaged temperature switch on RCIC turbine coupling and bearing caused an erroneous high temperature annunciation in the control room. Consequently, RCIC operability was not proven within the required time limit.
2. An engineering evaluation revealed that this event was caused primarily by a 4.5 MA ground fault at the negative terminal of station battery 2A. Ground faults as well as voltage transients (due to control switching) caused an overvoltage condition which exceeded the maximum voltage that the existing RCIC steam inlet valve time delay relay (2E51-K51) could withstand. The ground fault and voltage transients caused the RCIC steam inlet valve time delay relay to time erratically and open the RCIC steam supply valve (2E51-F045) which caused RCIC to inadvertently start.
3. This event was caused by personnel error, because a Design Change Request was not initiated prior to modification of plant equipment.

Immediate Corrective Action:

1. The RCIC turbine coupling and bearing temperature indicator (1E51-N753) was removed and a direct reading local thermometer was temporarily installed.
2. The excessively sensitive time delay relay was replaced with a less sensitive model per a design change request. RCIC was satisfactorily functionally tested per HNP-2-3405 on 07/12/83. The station service battery ground is no longer present.
3. Responsible personnel have been counseled concerning the importance of complying with administrative controls.

Supplemental Corrective Action:

1. A standing order was issued per the "PLANT OPERATING ORDERS" procedure (HNP-12) on 07/09/83. This requires a visual check of bearing temperature within 10 minutes after any RCIC start, and every 30 minutes thereafter while it is running.
2. No supplemental action is required.
3. No supplemental corrective action was taken. |

Scheduled (future) corrective action:

1. A new RCIC turbine coupling end bearing temperature indicator will be installed when it is received on site.
2. No future corrective action is scheduled.
3. A Design Change Request is in the writing process. |

Action to prevent recurrence (if different from corrective actions):

1. N/A
2. N/A
3. N/A |

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Georgia Power

Edwin I. Hatch Nuclear Plant

August 18, 1983
GM-83-807

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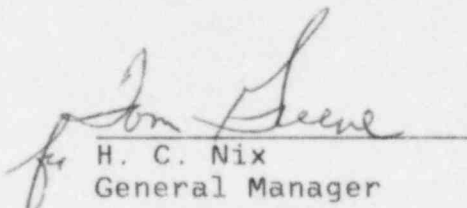
U.S. NRC REG.-II
ATLANTA, GA.

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
Suite 3100
101 Marietta Street
Atlanta, Georgia 30303

ATTENTION: Mr. James P. O'Reilly

Attached is Licensee Event Report No. 50-366/1983-050, Rev. 1. This report is required by Hatch Unit 2 Technical Specifications Section 6.9.1.9.c.


H. C. Nix
General Manager

see
HCN/SBT/djs

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