

**LICENSEE EVENT REPORT**

UPDATE REPORT

PREVIOUS REPORT DATE 4-28-83

CONTROL BLOCK: 

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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7	8	LICENSEE CODE						14	15	LICENSE NUMBER										25	26	LICENSE TYPE					30	57	CAT	58

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7	8			60	61	DOCKET NUMBER										68	69	EVENT DATE					74	75	REPORT DATE					80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | On 4/18/83, during performance of the In-Service Inspection (ISI)

0	3	Ultrasonic examination as per IEB 83-02, personnel noted cracklike
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04 | indications on the recirculation system's (2B31) endcap-to-manifold

05 | weld heat affected zone of weld 2B31-22BM-4. This event is contrary to

06 | Tech. Specs. section 3.4.8. ACTION a. The health and safety of the

07 | public were not affected by this non-repetitive event.

08 | \_\_\_\_\_ 8

7 8 9

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

0 9 C B 11 E 12 C 13 P I P E X X 14 E 15 Z 16

9 10 11 12 13 14 15 16 17 18 19 20

(17) LER/RO REPORT NUMBER

EVENT YEAR

SEQUENTIAL REPORT NO.

OCCURRENCE CODE

REPORT TYPE

REVISION NO.

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X	(18)	Z	Z	b b b b	Y	N	N	G O 8 O
33	34	35	36	37 40	41	42	43	44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | Inter-Granular Stress Corrosion (IGSC) has been considered as the most

1 1 | probable cause. The indications were evaluated during the last Unit 2

1	2
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 refueling outage. All the indications that needed repair were repaired

1 2 | before startup. 8 welds will be examined in an outage beginning no

1 4 | later than 12/31/83.

7	8	9	30	32
FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION

1	5	H	(28)	0	0	0	(29)	NA	B	(31)	In-Service Inspection
7	8	9		10	11	12	13		45	46	80

ACTIVITY		CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY (35)		LOCATION OF RELEASE (36)	
1	6	7	(33)	7	(34)	NA		NA	

PERSONNEL EXPOSURES  
NUMBER 395

NUMBER			TYPE	DESCRIPTION
1	7	000	(37) Z	(38) NA

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	2	3	4	5	6
		40			MA

7 8 9 11 12 NA  
LOSS OF OR DAMAGE TO FACILITY (43) 8309070296 830825  
FE22

TYPE		DESCRIPTION	
<u>1</u>	<u>9</u>	<u>Z</u>	(42) NA
2	8	9	10

PDR ADUCK 05000388  
S PDR

[illegible][illegible]

NAME OF PREPARER S. B. Tipps

PHONE: (912) 367-7851

NARRATIVE REPORT  
FOR LER 50-366/1983-023, Rev. 1  
UPDATE REPORT - PREVIOUS REPORT DATE 4/28/83

LICENSEE : GEORGIA POWER COMPANY  
FACILITY NAME : EDWIN I. HATCH  
DOCKET NUMBER : 50-366

Tech. Specs. section(s) which requires report:

this LER is required by Tech. Specs. section 6.9.1.8.c.

Plant conditions at the time of the event(s):

This event was discovered on 4/18/83, with the Unit in cold shutdown for refueling.

Detailed description of the event(s):

On 4/18/83, while performing the In-Service Inspection (ISI) ultrasonic examination of the recirculation system's welds as required by IEB 83-02, personnel noted crack-like indications in the endcap-to-manifold weld heat affected zone of weld 2B31-2BM-4.

Consequences of the event(s):

A LCO was initiated as per the requirements of Tech. Specs. section 3.4.8, ACTION a. The health and safety of the public were not affected.

Status of redundant or backup subsystems and/or systems:

The recirculation system does not have a redundant/backup system.

Justification for continued operation:

The indications were evaluated during the last Unit 2 refueling outage. All the indications that needed repair were repaired prior to startup.

If repetitive, number of previous LER:

This event is non-repetitive.

Impact to other systems and/or Unit:

This condition does not directly affect any other Unit 2 system; this event does not directly affect the operability status of Unit 1. However, ISI examinations were conducted on Unit 1 as per IEB 82-03 during the 1982-1983 refueling outage and all weld discrepancies were evaluated and repairs were made as necessary (Refer to LER 50-321/1982-089, Rev. 2).

Cause(s) of the event(s):

Inter-Granular Stress Corrosion (IGSC) has been considered as the cause of the crack indications.

Immediate Corrective Action:

No corrective action has been made. The cause and corrective action is presently being evaluated. All the indications that needed repair were repaired before startup.

Supplemental Corrective Action:

There is no supplemental corrective action required.

Scheduled (future) corrective action:

Upon completion of the evaluation, the necessary repairs were made and the system's component structure was functionally tested satisfactorily. A final stress analysis and fracture mechanics report (NED-83-326) was issued upon completion of the repair and before the plant returned to power operation. 8 welds have been chosen to be examined during an outage scheduled to begin no later than December 31, 1983. The subject welds will be examined to determine the relative stability of crack growth.

Action to prevent recurrence (if different from corrective actions):

The future corrective action should be sufficient to prevent recurrence of this event.

Georgia Power Company  
Post Office Box 439  
Baxley, Georgia 31513  
Telephone 912 367-7781  
912 537-9444

USNRC REGION II  
ATLANTA, GEORGIA

83 SEP 1 8:07  
Georgia Power

Edwin I. Hatch Nuclear Plant

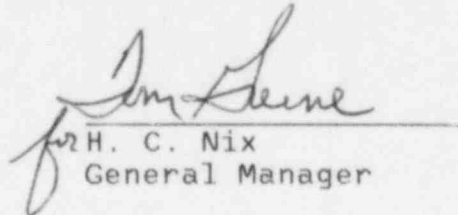
August 25, 1983  
GM-83-834

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region II  
Suite 3100  
101 Marietta Street  
Atlanta, Georgia 30303

ATTENTION: Mr. James P. O'Reilly

Attached is Licensee Event Report No. 50-366/1983-023, Rev. 1. This report is required by Hatch Unit 2 Technical Specifications Section 6.9.1.8.c.

  
H. C. Nix  
General Manager

861  
HCN/GBT/djs

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