

PREVIOUS REPORT DATE 5/19/83

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

0	1
7	8

REPORT SOURCE

L	6	0	5	0	0	0	3	6	6	7	0	4	2	8	8	3	8	0	8	1	8	8	3	9
60	61	DOCKET NUMBER						68	69	EVENT DATE						74	75	REPORT DATE						80

0 2 During performance of the "SAFETY/RELIEF VALVE POSITION, PRIMARY AND
0 3 SECONDARY INDICATORS F.T.&C" procedure, 2 safety/relief valve position
0 4 primary indicators (2B21-N301A&C) were found to actuate at 91 psig and
0 5 90.5 psig, respectively. Tech. Specs. section 4.4.2.b required that the
0 6 indicators actuation setpoint be 85+ 5 psig. The redundant safety/relief
0 7 valve position secondary indicators were operable. The health and
0 8 safety of the public were not affected by this non-repetitive event.

09		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE			
I B		E		Z		I N S T R U				S		Z					
9 10		11		12		13 14 15 16 17 18				19		20					
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.									
8 3		0 2 9		0 3		X		1									
21 22		23 24 25 26		27 28 29		30 31		32									
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
X C		Z		Z		0 0 0 0		Y		N		N		G 0 8 0			
33 34		35		36		37 38 39 40		41		42		43		44 45 46 47			

1 0 The cause of this event has been attributed to setpoint drift. Original

1 1 plans called for replacement of these switches prior to startup;

1 2 however, amendment number 33 to the Tech. Specs. was issued on 06/29/83.

1 3 This amendment changed section 4.4.2.b to 4.4.2.1.b, and the tolerance

1 4 to +15 psig and -5 psig. Thus, the switches were operable.

8 9
FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

1 5 [H] (28) 0 0 0 (29) NA [B] (31) Surveillance Test

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 [Z] (33) [Z] (34) NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PERSONNEL EXPOSURES
NUMBER TYPE DESCRIPTION (39)

1 7 0 0 0 (37) [Z] (38) NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PERSONNEL INJURIES
NUMBER DESCRIPTION (41)

1 8 0 0 0 (40) NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

LOSS OF OR DAMAGE TO FACILITY
TYPE DESCRIPTION (43)

1 9 [Z] (42) NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PUBLICITY
ISSUED DESCRIPTION (45)

2 0 [N] (44) NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

8309060286 830818
PDR ADOCK 05000366
S PDR

NRC USE ONLY

PHONE: (912) 367-7851

NARRATIVE REPORT
FOR LER 50-366/1983-029, Rev. 1
UPDATE REPORT - PREVIOUS REPORT DATE 5/19/83

LICENSEE : GEORGIA POWER COMPANY
FACILITY NAME : EDWIN I. HATCH
DOCKET NUMBER : 50-366

Tech. Specs. section(s) which requires report:

This 30-day LER is required by Tech. Specs. section 6.9.1.9.b. due to the event's showing that the unit was not meeting the requirements of Tech. Specs. section 4.4.2.1.b.

Plant conditions at the time of the event(s):

This event was discovered on 4/28/83 with the unit shutdown for refueling.

Detailed description of the event(s):

On 4/28/83, while performing the "SAFETY/RELIEF VALVE POSITION, PRIMARY AND SECONDARY INDICATORS F.T.&C" procedure (HNP-2-3820), 2 safety/relief valve position primary indicators were found to be out of Tech. Specs. tolerance. Tech. Specs. section 4.4.2.b. required the safety/relief valve position primary indicators (i.e., tail pipe pressure switches) to be operable with the setpoint to be 85 ± 5 psig. Tailpipe pressure switches 2B21-N301A and 2B21-N301C were found to actuate at 91 psig and 90.5 psig, respectively.

Consequences of the event(s):

Plant operation was not affected as the unit was shutdown for refueling when the event was discovered. The health and safety of the public were not affected by this event.

Status of redundant or backup subsystems and/or systems:

The redundant safety/relief valve position secondary indicators were operable as shown by the satisfactory completion of HNP-2-5264, "GENERAL ELECTRIC TYPE H.G. MULTIPOINT RECORDER" calibration procedure on 4/21/83.

Justification for continued operation:

None required because the unit was shutdown for refueling when the event was discovered.

If repetitive, number of previous LER:

This event is non-repetitive.

Impact to other systems and/or Unit:

This event had no impact on any other Unit 2 system or on Unit 1.

Cause(s) of the event(s):

The cause of this event is due to setpoint drift.

Immediate Corrective Action:

No immediate corrective action was taken because the unit was shutdown for refueling.

Supplemental Corrective Action:

The site originally planned to replace these switches prior to startup; however, Tech. Specs. amendment number 33 was issued on 06/29/83. This amendment changed old section number 4.4.2.b to 4.4.2.1.b, and the setpoint was changed to "85 psig, with an allowable tolerance of +15 psig and -5 psig". Thus, the switches were operable.

Scheduled (future) corrective action:

None was required.

Action to prevent recurrence (if different from corrective actions):

None

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Téléphone 912 367-7781
912 537-9444

USNRC REGION II
ATLANTA, GEORGIA

83 AUG 29 09:28



Georgia Power

Edwin I. Hatch Nuclear Plant

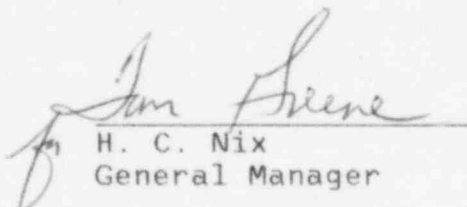
August 18, 1983
GM-83-799

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
Suite 3100
101 Marietta Street
Atlanta, Georgia 30303

ATTENTION: Mr. James P. O'Reilly

Attached is Licensee Event Report No. 50-366/1983-029, Rev. 1. This report is required by Hatch Unit 2 Technical Specifications Section 6.9.1.9.b.


H. C. Nix
General Manager

HCN/SBT/djs

xc: R. J. Kelly
G. F. Head
J. T. Beckham, Jr.
P. D. Rice
K. M. Gillespie
S. B. Tipps
R. D. Baker
Control Room
Document Control

ORIGINAL COPY

Te 22
11