

CONTROL BLOCK

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 ALBRF100-000000-000411114

CONT  
01 REPORT SOURCE L505000259072983082683

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During normal operation, while performing SI4.9.A.4.c(unit 1 & 2), 2 re-

03 lays on 4KV sht.bd. A and one relay on bd.C were found to operate between

04 3942-3950V. Max. allowable setpoint specified by T.S.Table 4.9.A.4.c is

05 3940V. There was no effect on public health and safety. T.S.3.9.B.11.b

06 allows operation for 10 days with a degraded voltage relay channel inop.

07 and T.S.3.9.B.11.c allows operation for 15 days with a degraded voltage

08 relay inop. Loss-of-voltage relay channel was available and operable.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE

EB11 B12 A13 RELAYX15 D16 Z16

17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

83 048 03 L 0

33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49

EX14 Z20 Z21 0000 Y23 Y24 L25 B455

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The relay's setpoints had apparently drifted up due to increases in

11 ambient temperature since their last calibration on 6/27/83. The Gould-

12 Brown Boveri Type ITE 27/59H relays were immediately recalibrated and re-

13 turned to service. Recurrence control is being evaluated and will be

14 addressed in a followup report to BFRO-50-259/8213 by December 1, 1983.

15 E25 09629 NA B31 Surveillance Test

16 Z31 Z34 NA NA

17 00037 Z38 NA

18 00040 NA

19 Z42 NA

20 N44 NA

8309060218 830826  
PDR ADOCK 05000259  
S PDR

NRC USE ONLY

NAME OF PREPARED

Walter Christopher

PRINT

(205)729-0889

LER SUPPLEMENTAL INFORMATION

BFRO-50-259 / 83048 Technical Specification Involved 3.9.B.11.b and c

Reported Under Technical Specification 6.7.2.b.(2)\* Date Due NRC 8/28/83

Event Narrative:

Unit 1 was in a refueling outage; unit 2 was operating normally at 96 percent power; unit 3 was operating normally at 97 percent power. Only units 1 and 2 were affected by this event. While performing Surveillance Instruction 4.9.A.4.c (Auxiliary Electrical Equipment undervoltage Relay Calibration for 4-KV Shutdown Boards), the following relay's setpoints were found out of tolerance: 4-KV shutdown board A, relay 27-211-1B drifted to 3950 volts and relay 27-211-1C drifted to 3942 volts; and 4-KV shutdown board C, relay 27-211-3A drifted to 3944 volts. Technical Specification Table 4.9.A.4.c specifies an allowable setpoint range of 3900-3940 volts. There was no effect on public health and safety. On each 4-KV shutdown board there are 3 degraded voltage relays. Two degraded voltage relays were available and operable on board C while only one relay was available and operable on board A. The loss-of-voltage relay channels were available and operable on all four 4-KV shutdown boards. Also, this drift was in a conservative upward direction. The degraded voltage relays were immediately recalibrated and returned to service within the time limits as specified by Technical Specifications 3.9.B.11.b and c. The relays are Gould-Brown Boveri type ITE 27-59H. The relay failures are due to setpoint drift which is believed to have been caused by ambient temperature increases since their last calibration. Recurrence control is being evaluated and will be addressed in a followup report to BFRO-50-259/82013 by December 1, 1983.

\* Previous Similar Events:

BFRO-50-259/82013, 82028, 82033, 82050, 82075, 82086  
BFRO-50-296/82032, 83040

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRP

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

83

August 26, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW  
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET  
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE  
REPORT BFR0-50-259/83048

The enclosed report provides details concerning degraded voltage relays  
found with trip setpoint above the allowable setting. This report is  
submitted in accordance with Browns Ferry unit 1 Technical Specification  
6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Records Center  
Institute of Nuclear Power Operations  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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