

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT
/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/8/0/5/8/3/ (8) /0/8/2/4/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On August 5, 1983, with Unit 1 in Mode 1, the outer personnel hatch door was /
/0/3/ / determined to have an unacceptable leak rate. This event is contrary to T.S. /
/0/4/ / 3.6.1.3 and reportable pursuant to T.S. 6.9.1.9.b. Because the inner personnel /
/0/5/ / hatch door was properly sealed, the health and safety of the general public were /
/0/6/ / not affected. There are no generic implications associated with this event. /
/0/7/ /
/0/8/ /

| SYSTEM CODE | CAUSE CODE | CAUSE SUBCODE | COMPONENT CODE | COMP. SUBCODE | VALVE SUBCODE | |
|----------------|---------------|------------------|--------------------------|--------------------|------------------|-----------------|
| /0/9/ | /S/A/ (11) | /E/ (12) | /B/ (13) | /P/E/N/E/T/R/ (14) | /A/ (15) | /Z/ (16) |
| | LER/RO | EVENT YEAR | SEQUENTIAL REPORT NO. | OCCURRENCE CODE | REPORT TYPE | REVISION NO. |

(17) REPORT NUMBER /8/3/ /-/ /0/5/5/ / / /0/3/ /L/ /-/ /0/
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
/A/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /Y/ (24) /A/ (25) /C/3/1/0/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The outer door seal was found to be worn due to normal use. This seal was re- /
/1/1/ / placed. The outer personnel hatch door was satisfactorily retested within 24 /
/1/2/ / hours as required by the applicable Technical Specification. /
/1/3/ /
/1/4/ /

| FACILITY STATUS | %POWER | OTHER STATUS | METHOD OF DISCOVERY | DISCOVERY DESCRIPTION (32) |
|---------------------------------------|-----------------------|-------------------------|--------------------------|-----------------------------------|
| /1/5/ | /F/ (28) | /0/9/4/ (29) | / NA / (30) | /B/ (31) / Surveillance Testing / |
| ACTIVITY RELEASED | CONTENT OF RELEASE | AMOUNT OF ACTIVITY (35) | LOCATION OF RELEASE (36) | |
| /1/6/ | /Z/ (33) | /Z/ (34) | / NA / | / NA / |
| PERSONNEL EXPOSURES NUMBER | TYPE | DESCRIPTION (39) | | |
| /1/7/ | /0/0/0/ (37) | /Z/ (38) | / NA / | / |
| PERSONNEL INJURIES NUMBER | DESCRIPTION (41) | | | |
| /1/8/ | /0/0/0/ (40) | / NA / | / | / |
| LOSS OF OR DAMAGE TO FACILITY TYPE | DESCRIPTION (43) | | | |
| /1/9/ | /Z/ (42) | / NA / | / | / |
| ISSUED | DESCRIPTION (45) | | | |
| /2/0/ | /N/ (44) | / NA / | / | / |

NRC USE ONLY

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

Vepco

USNRC REGION II
ATLANTA, GEORGIA

VIRGINIA ELECTRIC AND POWER COMPANY

83 AUG 29 P 2:04
NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

August 24, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-124
NO/RST: 11
Docket No. 50-338
License No. NPF-4

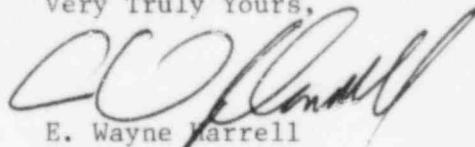
Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

| Report No. | Applicable Technical Specifications |
|------------------|-------------------------------------|
| LER 83-055/03L-0 | T.S. 6.9.1.9.b |

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,


E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
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U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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