

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/9/ (7) /0/7/3/1/8/3/ (8) /0/8/2/4/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On July 31, 1983, the Emergency Condensate Storage Tank dropped below the T.S. /
/0/3/ / minimum volume of 110,000 gallons when the Auxiliary Feedwater Pumps were in /
/0/4/ / operation following a turbine trip reactor trip. All equipment performed as /
/0/5/ / designed. The health and safety of the public were not affected. This event is /
/0/6/ / contrary to T.S. 3.7.1.3 and reportable pursuant to T.S. 6.9.1.9.b. There are no /
/0/7/ / generic implications of this event. /
/0/8/ /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/	/W/F/ (11)	/X/ (12)	/Z/ (13)	/Z/Z/Z/Z/Z/Z/ (14)	/Z/ (15)
	LER/RO	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE
	REPORT NUMBER				REVISION NO.

(17)

	/8/3/	/-/	/0/6/4/	/	/0/3/	/L/	/-/	/0/
ACTION	FUTURE	EFFECT	SHUTDOWN	ATTACHMENT	NPRD-4	PRIME COMP.	COMPONENT	
TAKEN	ACTION	ON PLANT	METHOD HOURS	SUBMITTED	FORM SUB.	SUPPLIER	MANUFACTURER	

/X/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /N/ (24) /Z/ (25) /Z/9/9/9/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The Auxiliary Feedwater Pumps received an auto start signal due to a Lo-Lo steam /
/1/1/ / generator level signal following the reactor trip; a usual situation caused by /
/1/2/ / void collapse in the steam generator. Since the Auxiliary Feedwater Pumps take /
/1/3/ / suction from the Emergency Condensate Tank the level subsequently fell below /
/1/4/ / normal. Level was restored within the time requirements of T.S. 3.7.1.3. /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/	/G/ (28)	/0/0/0/ (29)	/ NA / (30)	/A/ (31) / Operator Observation /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/	/Z/ (33)	/Z/ (34)	/ NA /

PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
/1/7/	/0/0/0/ (37)	/Z/ (38)

PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
/1/8/	/0/C/0/ (40)

LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)
/1/9/	/Z/ (42)

PUBLICITY ISSUED	DESCRIPTION (45)
/2/0/	/N/ (44)

NRC USE ONLY

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

8309060125 830824
PDR ADOCK 05000339
S PDR

Vepco

USNRC REGION II
ATLANTA, GEORGIA

83 AUG 29 12:04
VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION
P. O. BOX 402
MINERAL, VIRGINIA 23117

August 24, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-123
NO/CLF: 11
Docket No. 50-339
License No. NPF-7

Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No.

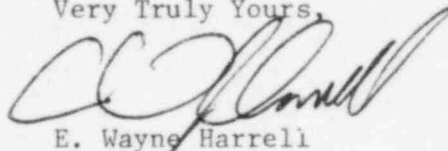
Applicable Technical Specifications

LER 83-064/03L-0

T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,


E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
016 Phillips Bldg.
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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