

TABLE OF CONTENTS

	<u>Revision</u>	<u>Date</u>
<u>1.0 CLASSIFICATION & ASSESSMENT</u>		
1.1 Initial Classification	5	07-01-83
1.2 Plant Status	1	07-01-83
1.3 Estimation of Source Term	6	07-01-83
1.4 Radiological Dose Evaluation	7	05-27-83
1.5 Protective Action Evaluation	4	09-10-82
1.6 Radioiodine Blocking & Thyroid Exposure Accounting	1	02-26-82
1.7 Evaluation of Core Damage	2	07-01-83
1.8 Emergency Off-Site Dose Estimations	3	07-01-83
<u>2.0 UNUSUAL EVENT IMPLEMENTING PROCEDURES</u>		
2.1 Unusual Event - Immediate Actions	1	09-10-82
2.2 Unusual Event - Plant and Company Personnel Notification	1	07-01-81
2.3 Unusual Event - Off-Site Agency Notification	3	05-27-83
<u>3.0 ALERT IMPLEMENTING PROCEDURES</u>		
3.1 Alert - Immediate Actions	2	09-10-82
3.2 Alert - Plant & Company Personnel Notification	1	07-01-81
3.3 Alert - Off-Site Agency Notification	2	05-27-83
<u>4.0 SITE EMERGENCY - IMPLEMENTING PROCEDURES</u>		
4.1 Site Emergency - Immediate Actions	2	09-10-82
4.2 Site Emergency - Plant & Company Personnel Notification	1	07-01-81
4.3 Site Emergency - Off-Site Agency Notification	1	09-10-82
<u>5.0 GENERAL EMERGENCY - IMPLEMENTING PROCEDURES</u>		
5.1 General Emergency - Immediate Actions	2	09-10-82
5.2 General Emergency - Plant & Company Personnel Notification	1	07-01-81
5.3 General Emergency - Off-Site Agency Notification	1	09-10-82
<u>6.0 EVACUATION</u>		
6.1 Limited Plant Evacuation	2	01-28-83
6.2 Plant Evacuation	2	01-28-83
6.3 Exclusion Area Evacuation	1	01-28-83
6.4 Energy Information Center Evacuation	0	03-31-81
6.5 TSC & OSC Activation	2	07-01-83

		<u>Revision</u>	<u>Date</u>
7.0	<u>CHEMISTRY & HEALTH PHYSICS RESPONSE & PREPAREDNESS</u>		
7.1	<u>Internal Chem & HP Group Personnel Notification/ Initial Response</u>		
7.1.1	Chem & HP Group Personnel Notification & Initial Response when Chem & HP Personnel are On-Site	2	04-30-82
7.1.2	Chem & HP Group Personnel Notification & Initial Response when Chem & HP Personnel are Off-Site	1	03-17-82
7.1.3	HP Protective Actions by Operations Personnel Prior to Arrival of Chem & HP Group Personnel	1	05-15-81
7.2	<u>Health Physics Facility Activation</u>		
7.2.1	Activation of HP Facilities at Site Boundary Control Center	2	03-17-82
7.2.2	Activation of HP Facilities at Operations Support Center	2	05-27-83
7.2.3	DELETED		
7.2.4	Health Physics Communications	2	05-27-83
7.2.5	Control & Use of Vehicles	1	03-17-82
7.3	<u>Radiological Surveys</u>		
7.3.1	Airborne Sampling & Direct Dose Rate Survey Guidelines	3	03-17-82
7.3.2	Post-Accident Sampling & Analysis of Potentially High Level Reactor Coolant	7	07-15-83
7.3.3	Post-Accident Sampling of Contain- ment Atmosphere	6	07-15-83
7.3.4	Movement of Required Chemistry Equip- ment & Material to the Technical Support Center Counting Room & Mini-Laboratory	0	12-30-81
7.4	<u>Emergency Equipment</u>		
7.4.1	Routine Check, Maintenance, Cali- bration & Inventory Schedule for Health Physics Emergency Plan Equipment	7	05-27-83
7.4.2	Emergency Plan Equipment Routine Check, Maintenance & Calibration Instructions	5	05-27-83
7.4.3	Use of Baird Model 530 Single Channel Iodine Spectrometer to Determine Airborne Iodine Activities	1	05-15-81

	<u>Revision</u>	<u>Date</u>
7.4.3.1 Use of Canberra Model 3100 Series 30 Multichannel Analyzer to Determine Airborne Iodine Activities	0	02-26-82
7.4.4 AMS-2 Air Particulate, Iodine & Noble Gas Sampler/Detector	0	03-31-81
8.1 <u>Personnel Assembly & Accountability</u>	3	01-28-83
9.1 <u>Security</u>	0	03-31-81
10.0 <u>Firefighting</u>	0	03-31-81
11.0 <u>FIRST AID & MEDICAL CARE</u>		
11.1 On-Site First Aid Assistance	3	09-10-82
11.2 Injured Person's Immediate Care	1	05-15-81
11.3 Hospital Assistance	3	01-28-83
11.4 Personnel Decontamination	0	01-29-82
12.0 <u>REENTRY & RECOVERY PLANNING</u>		
12.1 Reentry Procedures for Emergency Operations	1	03-17-82
12.2 Personnel Exposure & Search & Rescue Teams	2	04-30-82
12.3 Recovery Planning	0	03-31-81
12.4 Personnel Monitoring Exposure Guidelines . .	0	01-29-82
13.0 <u>PRESS</u>		
13.1 Crisis Communications	2	09-10-82
14.0 <u>COMMUNICATIONS</u>		
14.1 Testing of Communications Equipment	0	03-31-81
15.0 <u>TRAINING, DRILLS & EXERCISES</u>		
15.1 Employee Training	1	09-04-81
15.2 Off-Site Personnel Training	0	03-31-81
15.3 Drills & Exercises	2	04-30-82
15.4 Emergency Preparedness Review	0	09-10-82
16.0 <u>WISCONSIN ELECTRIC GENERAL OFFICE PROCEDURES</u>		
16.1 <u>Nuclear Engineering Section Notification & Response</u>	4	07-01-83

TABLE OF EPIP FORMS

<u>EPIP Form</u>	<u>Title</u>	<u>EPIP Procedure</u>
01	Emergency Plan Airborne Radiation Survey Record Site Boundary Control Center (01-83)	7.3.1
02	Emergency Plan Survey Record Site Boundary Control Center (01-83)	7.3.1
03	Dose Factor Calculations for Specific Noble Gas Analysis Results (03-81)	7.3.1
04	Status Report on Plant Systems & Controls for Affected Unit (01-83)	1.2
05	Worksheet for Status Report on Radiation Monitoring System for Unit (07-83)	1.2
06	Worksheet for Status Report on Radiation Monitoring System for Plant (07-83)	1.2
07	For X/Q Determination (09-82)	1.4
08	Estimated Whole Body & Thyroid Projected Doses (09-82)	1.4
09	Estimated Whole Body Dose Calculation Worksheet for Specific Noble Gas Releases (09-82)	1.4
10	Estimated Ground Deposition Calculation Worksheet for Particulate Radionuclide Releases (09-82)	1.4
10a	Estimated Population Dose (09-82)	1.8
11	Summary of Radiological Dose Evaluation Calculations (09-82)	1.4
12	Incident Report Form (09-82)	2.3, 3.3, 4.3, 5.3
13	Status Update Form (09-82)	2.3, 3.3 4.3, 5.3
14	Unused	
15	Unused	
16	Event Data Checklist (03-81)	5.3
17	Accounting Short Form (04-82)	8.1
18	Assembly Area Roster (03-81)	3.1
19	Drill/Exercise Scenario (03-81)	15.3
20	Drill/Observation Sheet (03-81)	15.3
21	Drill/Exercise Evaluation Report (03-81)	15.3
22	Plant & Company Emergency Call List (01-83)	Call List Tab
23	Offsite Agency Emergency Call List (01-83)	Call List Tab
24a	Site Boundary Control Center Emergency Plan Inventory Checklist (05-83)	7.4.1
24b	TSC, ESC, South Gate & OSC Emergency Plan Inventory Checklist (09-82)	7.4.1
24c	Emergency Plan Health Physics Supplies at Two Rivers Community Hospital Inventory Checklist (09-81)	7.4.1
24d	Control Room Emergency Plan Equipment Inventory Checklist (06-82)	7.4.1
24e	Emergency Vehicle Inventory Checklist (04-82)	7.4.1
24f	Emergency Plan First Aid Kit Inventory Checklist (09-82)	7.4.1

<u>EPIP</u> <u>Forms</u>	<u>Title</u>	<u>EPIP</u> <u>Procedure</u>
24g	Emergency Plan Burn Kit Inventory (09-82)	7.4.1
24h	Emergency Plan First Aid Room Inventory (09-82)	7.4.1
24i	Emergency Plan Stretcher Inventory (09-82)	7.4.1
24j	Emergency Trauma Kit Inventory Checklist (09-82)	7.4.1
25a	Emergency Vehicle Checklist (06-82)	7.4.2
25b	Monthly Health Physics Instrument & Air Sampler Functional Test Checklist (01-83)	7.4.2
25c	Quarterly Emergency Plan Checklist (05-83)	7.4.2
25d	Semi-Annual & Annual Emergency Plan Checklist (05-83)	7.4.2
26	Quarterly Communications Test (03-81)	14.1
27	Monthly Communications Test (01-83)	14.1
28	Emergency Plan Instrument Calibration Schedule (06-82)	7.4.2
29	Emergency Plan Counting Equipment & Frisker Calibration Schedule (06-82)	7.4.2
30	Reactor Coolant Post-Accident Sampling Analysis Report (07-83)	7.3.2
31	Containment Atmosphere Post-Accident Sampling Analysis Report (07-83)	7.3.3
32	Search & Rescue and Emergency Operations Checklist (04-82)	12.2
33	Estimation of Core Damage (09-82)	1.7
34	Calculation of Xe-133 Equivalent Release Rates (07-83)	1.8
35	Dose Calculations (09-82)	1.8
36	Unused	
37	Medical Assistance Call List (09-82)	11.1

POST-ACCIDENT SAMPLING AND ANALYSIS
OF POTENTIALLY HIGH LEVEL REACTOR COOLANT

1.0 INTRODUCTION

1.1 This procedure outlines the steps necessary to collect, handle and analyze a high level reactor coolant sample which could result from gross fuel failure.

1.2 Equipment List

Set up the following in the primary sample hood prior to collecting your sample:

- 1.2.1 The equipment detailed in Figures 2A and 2B, with the exception of the sample bomb.
- 1.2.2 One magnetic stirrer and one 50 ml poly beaker and a 50 ml beaker.
- 1.2.3 A pd/mv meter and pH probe.
- 1.2.4 A piston burette.
- 1.2.5 A lead brick wall of sufficient size to store residue from analysis.
- 1.2.6 Automatic pipets, continuously adjustable through 5 ml, or equivalent.
- 1.2.7 Remote handling tools located in the cabinet below the hood.
- 1.2.8 Prepare a 1.0 liter sidearm flask with a correctly sized solid stopper and rubber septum over the sidearm.
- 1.2.9 Gas syringe

The following equipment is also necessary for this procedure and need to be made ready.

- 1.2.10 The gas partitioner for H₂ analysis.

- 1.2.11 The special cart used for transport of the sample bomb.
- 1.2.12 Tools for connecting and disconnecting sample bomb; i.e., 11/16" open end wrench or equivalent.
- 1.2.13 Remote valve turning tool. This tool as well as those mentioned in Step 1.2.12 are necessary for sampling and should be taken along and placed on the sample bomb transport cart.

1.3 Reagents

The following reagents are also necessary and need to be prepared.

- 1.3.1 0.1N NaOH for boron. Obtain a supply from normal boron analysis.
- 1.3.2 Manitol for boron analysis.

1.4 Preliminary Steps

Initials

- 1.4.1 Standardize the pH meter. _____
- 1.4.2 Organize as much of the equipment as possible behind lead brick walls in an arrangement that allows for unobstructed view of all operations with the aid of the convex mirror. (See Figure 2.) _____
- 1.4.3 Put new rubber septum on gas bomb. _____
- 1.4.4 Check out and prime the piston burette with fresh 0.1N NaOH solution. _____

2.0 PRELIMINARY EVALUATION

NOTE: THIS EVALUATION SHALL BE COMPLETED PRIOR TO ANY ATTEMPT TO ENTER THE AUXILIARY BUILDING OR SAMPLE ROOM TO OBTAIN A REACTOR COOLANT SAMPLE UNDER EMERGENCY CONDITIONS.

2.1 Possible Indication of Fuel Damage

Some or all of the following would be present if fuel damage had occurred:

- 2.1.1 The letdown radiation monitor (R9) would be unusually high or offscale. _____

Initials

- 2.1.2 The Unit 2 failed fuel monitor (2RE-109) will be unusually high or offscale. _____
- 2.1.3 The Unit 1 containment radiation monitors (R11 and R12) or the Unit 2 containment radiation monitors (RE-211 and RE-212) would be unusually high or offscale. _____
- 2.1.4 The Unit 1 containment area monitors (R2 and R7) or the Unit 2 containment area monitors (RE-102 and RE-107) would be reading unusually high or offscale. _____
- 2.1.5 The auxiliary building stack monitor (R14) would show a significant increase due to auxiliary building airborne activity from the letdown and charging pump areas. _____
- 2.1.6 Evaluation of Sample Room Conditions
- a. The sample room area monitor (R6 or RE-106) and charging pump area monitor (R4 or RE-104) would give an indication of conditions in the auxiliary building and sample room. _____
 - b. After evaluation of the radiation monitoring system readouts, Health Physics will determine what airborne and radiation surveys would be appropriate before auxiliary building entry. _____
 - c. Verify the requirements for auxiliary building sample room entry, i.e., (1) RWP requirements, (2) clothing requirements, (3) respiratory requirements, and (4) dosimetry requirements including extremity dose monitoring requirements, and (5) health physics coverage requirements including timekeeping. _____

2.2 Possible Loss of Component Cooling

- 2.2.1 Verify that component cooling is still in service to the affected unit. Sample cannot be obtained without component cooling to sample room heat exchangers. _____

3.0 REACTOR COOLANT SAMPLING PROCEDURE

NOTE: THIS PROCEDURE SHALL NOT BE INITIATED UNTIL THE EVALUATIONS DISCUSSED IN SECTION 2.0 HAVE BEEN COMPLETED AND REVIEWED BY DUTY & CALL SUPERINTENDENT (COORDINATOR), DUTY HEALTH PHYSICS SUPERVISION AND THE DUTY SHIFT SUPERINTENDENT, AND THEIR APPROVAL HAS BEEN GRANTED.

3.1 Collecting a Pressurized Sample (Refer to Figure 1) Initials

NOTE: THE FOLLOWING STEPS WILL BE ACCOMPLISHED UNDER THE DIRECTION OF HEALTH PHYSICS SUPERVISION AND ONLY AFTER COMPLETING SECTION 1.0 OF THIS PROCEDURE.

3.1.1 The following steps (a through f) must be accomplished before opening the incontainment sample isolation valve 955 (Step 3.1.2) and the hot leg sample isolation valve 966C (Step 3.1.3). _____

- a. Verify that the demineralized water header pressure is approximately 100 to 120 psi. _____

NOTE: THIS STEP MAY BE DELETED IF REACTOR MAKEUP WATER IS USED FOR THE FLUSH.

- b. Proceed to the sample room and install the shielded sample bomb on the outside wall of the sample room using the fittings provided. _____

CAUTION: BEFORE REMOVING THE SWAGE LOCK CAPS TO INSTALL THE BOMB, OPEN VALVES 939, 940 AND 941 TO RELIEVE SYSTEM PRESSURE. CLOSE VALVES 939, 940 AND 941. PLACE A WASTE BUCKET DIRECTLY UNDER THE BOMB. USE A PAPER TOWEL SHIELD AND RUBBER GLOVES WHEN REMOVING CAPS.

- c. Verify that the demineralized water line is connected from valve 945 to the demineralized water manifold. Open demineralized H₂O valves 945, 947, and 948 and check for leaks on the bomb fittings. Shut valves 945, 947, and 948. _____

- d. Enter the sample room and close the following valves on the sample panel.

1. 961C - Normal hot leg sample bomb inlet. _____

2. 964C - normal hot leg sample bomb outlet. _____

Initials

3. 965C - normal hot leg sample bypass. _____
4. 968 - normal hot leg return valve. _____
5. 971 - normal hot leg sink sample valve. _____
- e. Notify the control room that the failed fuel monitor (RE-109) will be taken out-of-service for sampling; then close:
- 938B - detector RE-109 inlet valve _____
- 938A - detector RE-109 outlet valve _____
- f. Further verify that there is component cooling flow to the sample room heat exchangers by viewing the local flow indicator (FI-603) on the sample panel. _____
- g. Open the following valves on the sample panel.
1. 969A - sample system purge to volume control tank. _____
2. 956C - normal hot leg sample supply valve. _____
3. 990 - residual heat removal sample supply valve. _____
- h. Leave the sample room and position the following valves wide open. These valves are located on the sample room wall.
1. 939 - sample bomb inlet. _____
2. 940 - sample bomb bypass. _____
3. 941 - sample bomb outlet. _____

NOTE: FOR DRILLS AND TRAINING EXERCISES
OPEN VALVE 941 ONLY ONE-QUARTER TURN
TO ELIMINATE N-16 GAMMAS.

- 3.1.2 Open the incontainment hot leg sample isolation valve (valve 955) and the residual heat removal sample isolation valve (valve 959) by means of the switches located outside the No. 1 pipeway for Unit 1 or No. 4 pipeway for Unit 2.
-

CAUTION: IF THE AFFECTED UNIT IS ALREADY ON RHR, LEAVE THE AREA IMMEDIATELY AFTER OPENING VALVE 959 AS BACKGROUND RADIATION LEVELS WILL RISE SHARPLY.

If the valve No. 955 will not open because of containment isolation, perform the following steps (1 through 3).

1. Request that the Control Room reset the containment isolation signal.
 2. Turn the local control switch positions for valves 951, 953, and 955 to the "close" position.
 3. Turn the local control switch for valve 955 to the "open" position.
-

NOTE: SECTION 3.1.2 MUST BE ACCOMPLISHED BEFORE SAFETY INJECTION RECIRCULATION HAS BEGUN.

- 3.1.3 Leave the area and request control room to open the hot leg sample isolation valve (966C).
-
- 3.1.4 Verify sample flow by an increase in the radiation level.
-
- 3.1.5 After a minimum recirc time of 15 minutes, return to the sample station and using the remote valve operating tool, fully open valves 9B and 9A and 8A and 8B.
-
- a. Slowly and completely close valve 940.
 - b. Leave the primary auxiliary building.
-

NOTE: THE VALVE OPERATING TOOL SHOULD BE USED TO OPERATE ALL VALVES EXCEPT 945, 946, 947 AND 948 (FLUSH VALVES).

- 3.1.6 After 5 additional minutes, return to collect the sample. Close valves 9B and 9A and then valves 8B and 8A using the remote valve operating tool. Make note of the sample collection time.
-

NOTE: DO NOT DISCONNECT THE SAMPLE BOMB UNTIL SAMPLE FLOW IS SECURED AND THE DI FLUSH IS COMPLETE AS EVIDENCED BY REDUCTION IN RADIATION LEVELS.

- 3.1.7 Request control room to immediately close the hot leg sample isolation valve 966C.
-

NOTE: IT IS VERY IMPORTANT THAT THE HOT LEG SAMPLE ISOLATION VALVE 966C IS CLOSED PRIOR TO STARTING THE DI FLUSH. WAIT FOR CONFIRMATION FROM THE CONTROL ROOM.

4.0 SAMPLE LINE FLUSHING

- 4.1 Leave valve 939 open, and fully open valves 940 and 941. Open valves 945 and 946. Allow the lines to flush for at least 15 minutes.
-

NOTE: DO NOT REMAIN IN THE AREA OF THE SAMPLE STATION DURING THIS FLUSH.

- 4.2 After about 15 minutes return and measure radiation levels. If a Chemistry & Health Physics Supervisor determines that the levels are satisfactory, close whitey valve 946 and using the remote valve tool, close valve 939. Then open valve 947 and valve 948 and allow about a 15-minute DI flush.
-

- 4.3 After about 15 minutes, close valves 940 and 941 with the remote valve tool and then close valves 945, 947, and 948. Disconnect valve 945 from the demineralized water manifold and cap both ends. Disconnect the sample bomb from the fittings using a paper towel to prevent spraying. Remove the shielded sample bomb from its support. Remove excess liquid from the top and bottom bomb fittings with a syringe and dispose behind lead shielding. Replace the Swagelok caps on the wall fittings and on the bomb. Transport the bomb, remote valve tool and wrenches to the chemistry lab on a cart.
-

NOTE: AFTER DRILLS AND TRAINING EXERCISES, RETURN ALL EQUIPMENT AND VALVE LINEUPS TO NORMAL.

- 4.4 Enter the sample room and notify control room that the failed fuel monitor (RE-109) is to be returned to service and that the sample room valving is to be returned to normal.

4.4.1 Close Valves:

1. 961C - Normal hot leg sample bomb inlet _____
2. 964C - Normal hot leg sample bypass _____
3. 965C - Normal hot leg sample bypass _____
4. 971 - Normal hot leg sink sample valve _____
5. 939 - High level sample bomb inlet _____
6. 940 - High level sample bomb bypass _____
7. 941 - High level sample bomb outlet _____
8. 945, 946, 947, 948 - Demineralized water flush valves _____

NOTE: DISCONNECT VALVE 945 FROM THE DEMINERALIZED WATER MANIFOLD. ADVISE CONTROL TO REDUCE DEMINERALIZED WATER HEADER PRESSURE TO NORMAL.

4.4.2 Open Valves:

1. 938B - Failed fuel monitor RE-109 inlet _____
2. 938A - Failed fuel monitor RE-109 outlet _____
3. 956C - Normal hot leg sample supply valve _____
4. 968 - Normal hot leg return valve _____
5. 969A - Sample system purge to volume control tank _____
6. 966C - Hot leg sample isolation valve. _____

5.0 SEPARATION OF THE PRESSURIZED SAMPLE AND ANALYSIS OF THE GASEOUS AND LIQUID COMPONENT (Refer to Figures 2A and 2B)

5.1 Collecting the Gaseous Sample From the Pressurized Sample

- 5.1.1 Place the shielded sample bomb in the sample holder in the primary sample hood. _____

- 5.1.2 Connect the sample bomb to the shielded gas collection bomb by means of the fittings provided. Place lead bricks in the area of this connection for shielding. _____
- 5.1.3 Connect the valve manifold to the opposite end of the sample bomb and verify that valve 11 on the manifold is open. _____
- 5.1.4 Make sure that the vacuum line is attached to the gas collection bomb at the valve 1 location. Open valves 1 and 2. Secure valve 3. Evacuate the gas bomb and connecting lines. With vacuum still on, secure valve 1. Secure vacuum. _____
- 5.1.5 Before proceeding, make sure no inleakage has occurred into the gas bomb by observing the vacuum gauge reading. Using the remote valve tool, fully open valves 9A and 9B. Open valve 8A one-quarter turn. Crack open valve 8B and control degassing by throttling valve 8B. Allow the system to degas for 5 minutes. Check that valves 9A and 9B, 8A and 8B are fully open. Close valve 2. _____

NOTE: OBSERVE THE VACUUM GAUGE. THE VACUUM SHOULD DROP VERY SLOWLY. IF THE DROP IS TOO RAPID, CLOSE VALVE 8B SLIGHTLY. DROP SHOULD BE 5-10" HG/MIN.

5.2 Analysis of Gaseous Sample

5.2.1 Hydrogen

Use a syringe to draw a 1 cc sample. Use the injection port on the gas partitioner for this analysis. _____

5.2.2 Radioactive Noble Gas

Use a syringe to draw a 1/2 cc sample and inject this into the flask prepared in Section 1.2.8.* Allow 30 minutes for thermal mixing. Draw a 1 cc sample of this dilution and proceed as normal. _____

*Additional dilution should be performed if the contact reading is >1 mr/hour.

NOTE: SEE SECTIONS 7.0 AND 8.0.

5.3 Collecting the Liquid Sample From the Pressurized Sample

NOTE: IT IS EXTREMELY IMPORTANT TO VERIFY THAT VALVE 2 HAS BEEN CLOSED BEFORE PROCEEDING.

5.3.1 Open valve 3 slowly. Allow the liquid sample to drain into the 50 ml beaker. Direct a slow stream of air through the vent line on valve 3 if necessary to recover the total liquid sample. _____

5.3.2 Close valves 8A and 8B, 9A and 9B, and valve 3. _____

5.4 Analysis of Liquid Samples

5.4.1 Boron/pH Analysis

- a. Transfer a 5 ml sample using an automatic pipetter into a 50 ml poly beaker containing a stir bar. _____
- b. After transfer is complete, record the pH. _____
- c. Plug in the magnetic stirrer, add mannitol, and proceed with the boron analysis. _____

NOTE: IF THE PRIMARY SYSTEM HAS BEEN BORATED, 5 ML OR MORE OF TITRANT MAY BE NEEDED TO REACH AN ENDPOINT.

NOTE: AFTER DRILLS AND PRACTICE RUNS, THE BORON TITRATOR MUST BE FLUSHED WITH DEIONIZED WATER AND PUT IN DRY LAYUP. ALL ELECTRODES SHOULD BE PLACED IN LAYUP SO THEY ARE CONDITIONED FOR IMMEDIATE USE.

5.4.2 Chloride Analysis

NOTE: THE CHLORIDE ANALYSIS WILL BE PERFORMED BY WE LAB SERVICES USING THEIR ION CHROMATOGRAPHY ANALYZER.

Initials

5.4.3 Iodine Analysis and Gamma Scan

Using the specially prepared 2 cc syringe or adjustable automatic pipet, withdraw 0.3 cc (or less) of the sample from the poly beaker and inject this sample into a 1000 ml poly bottle containing demineralized water. Make additional dilutions in the same manner.* Count as normal. _____

*Additional dilution should be performed until the contact reading is <1 mr/hour.

NOTE: SEE SECTIONS 7.0 AND 8.0.

5.5 Reporting of Results

Complete and forward Reactor Coolant Post-Accident Sampling Analysis Report (EPIP-30).

6.0 SAMPLE RESIDUE

Place all sample residue in the specially prepared lead pig for disposal. _____

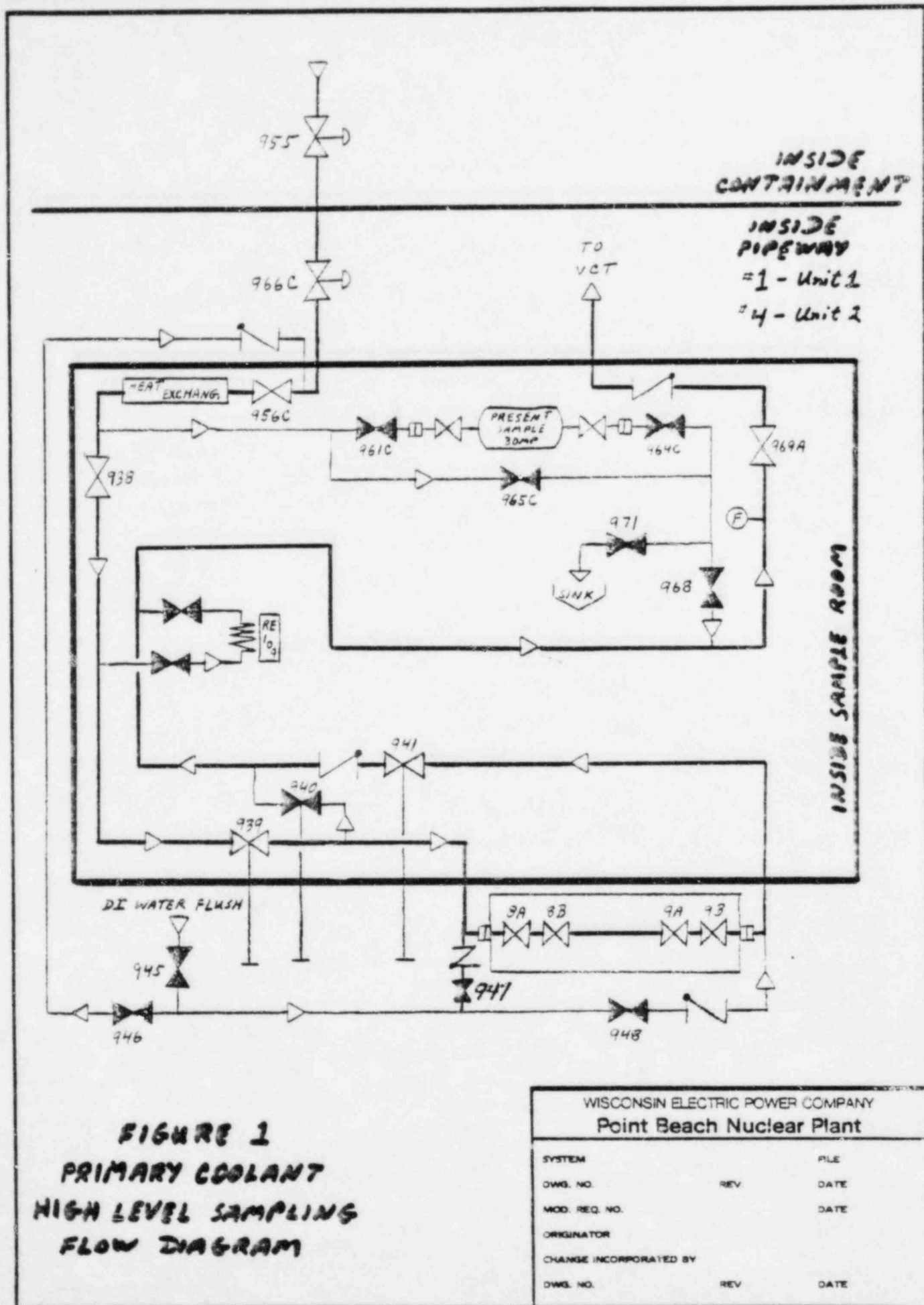
7.0 SAMPLES TAKEN TO KEWAUNEE NUCLEAR PLANT FOR COUNTING

Reference: Post-accident counting agreement with Wisconsin Public Service, Kewaunee Nuclear Plant.

Kewaunee Nuclear Plant does not utilize the 5 cc glass vial and 1 cc test tube geometries. Therefore, "normal" samples will have to be diluted and placed in one liter poly bottles if they are sent to the Kewaunee Nuclear Plant for analysis.

8.0 LABELING OF SAMPLES

Label all chloride, noble gas, iodine and gamma scan samples with all pertinent information such as: sample number, name of sample, date and time of sampling, sample volume and dilution(s).





CALCULATION SHEET

SHEET _____ OF _____

FILE No. _____

FIGURE 2 A

MADE BY _____ DATE _____

CHKD. BY _____ DATE _____

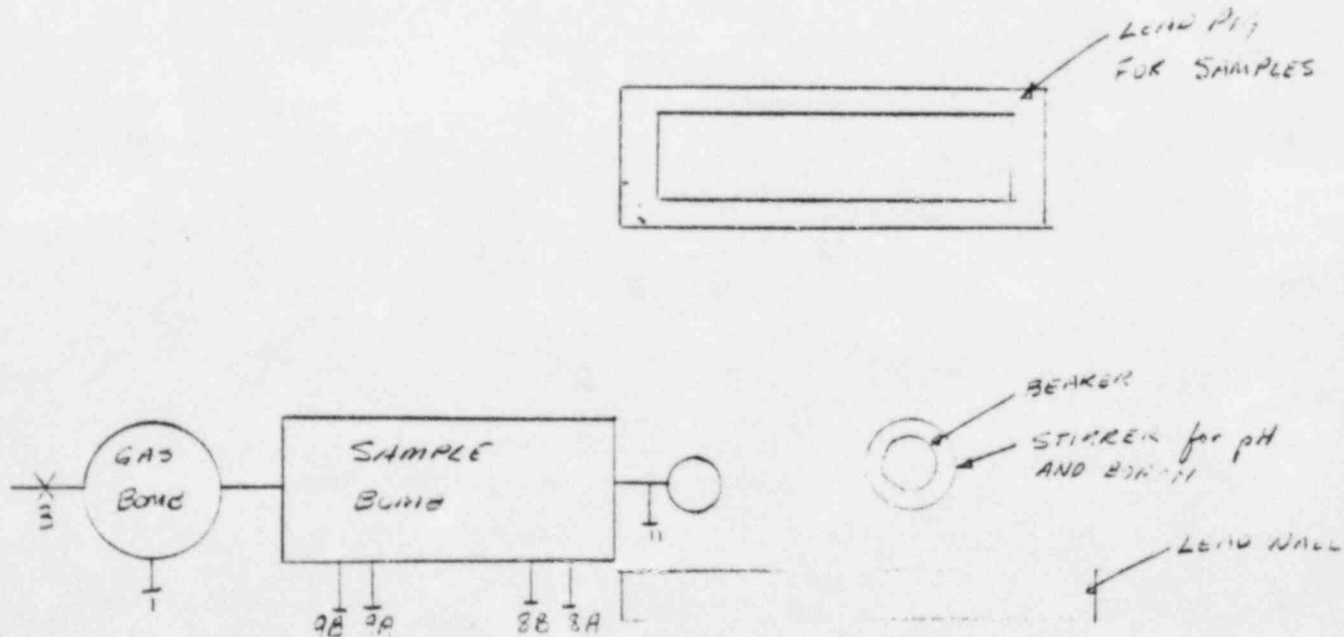
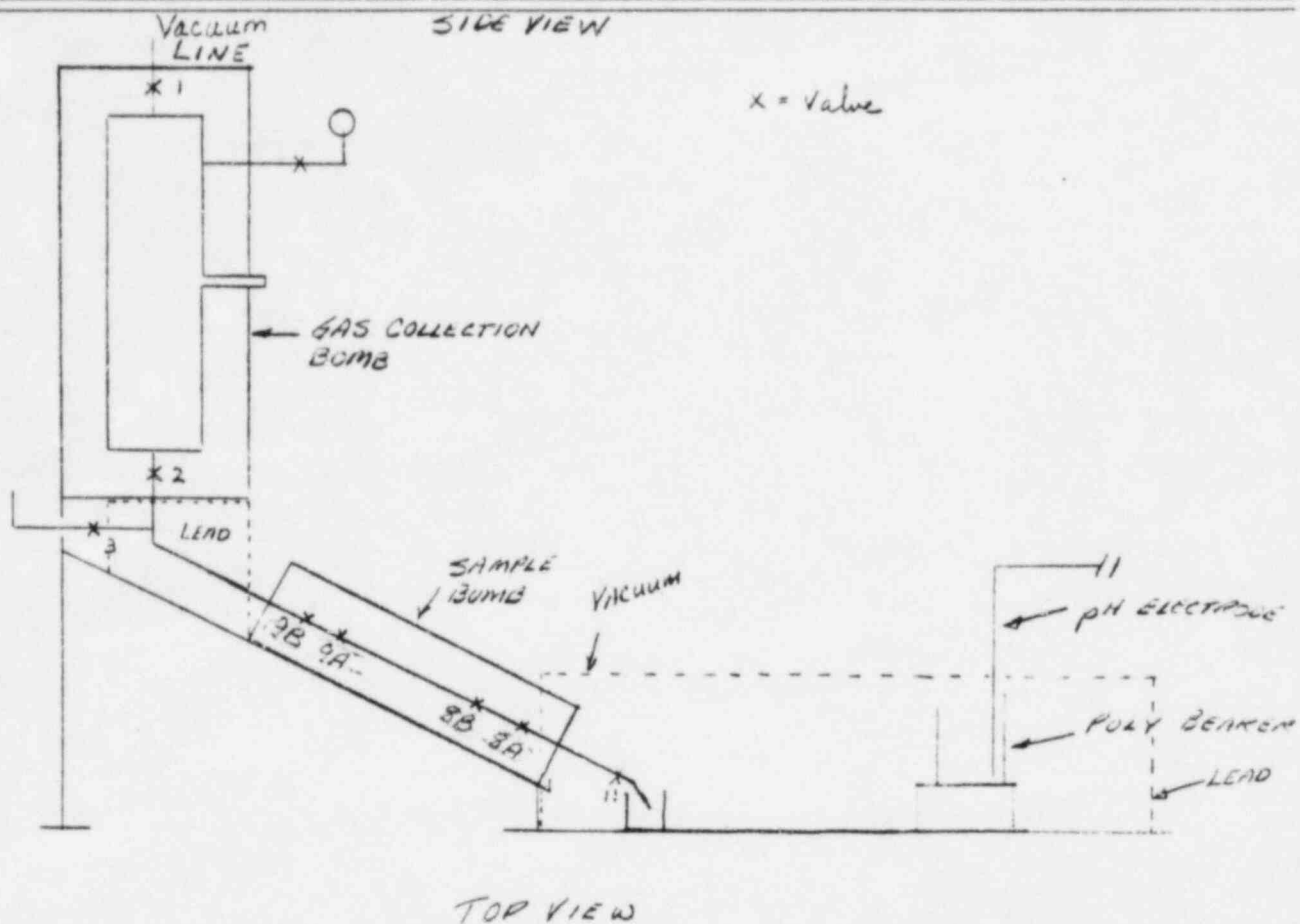




Figure 2 B

CALCULATION SHEET

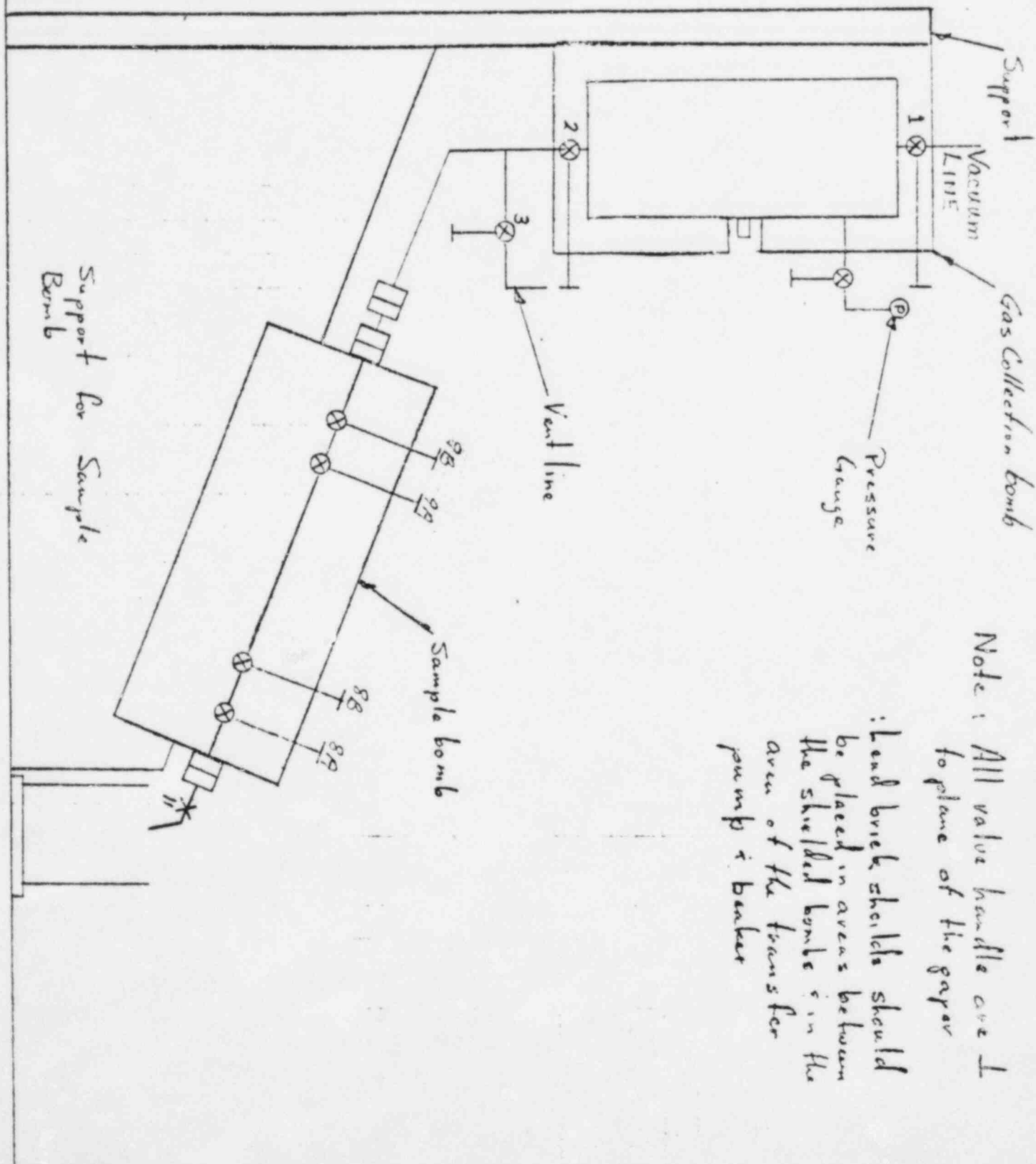
SHEET _____ OF _____

FILE No. _____

Sample Bomb Laboratory
Set up

MADE BY PJS DATE 1/3/90

CHKD. BY CHH DATE 1/3/90



Note: All valve handles are \perp to plane of the paper
Lead brick shields should be placed in areas between the shielded bombs & in the area of the transfer pump & Barker

POINT BEACH NUCLEAR PLANT

REACTOR COOLANT POST-ACCIDENT SAMPLING ANALYSIS REPORT

1.0 ANALYSIS OF GASEOUS SAMPLE

1.1 Hydrogen Analysis

- 1.1.1 Scale used on gas partitioner _____
- 1.1.2 Millivolt reading on chart recorder _____
- 1.1.3 % hydrogen from calibration curve _____
- 1.1.4 Volume of sample bomb in ml _____
- 1.1.5 Atmospheric pressure (mm/Hg) _____
- 1.1.6 cc/Kg hydrogen in coolant* (STP) _____

$$\begin{aligned} \text{*cc/Kg H}_2 \text{ (STP)} &= \frac{(1.1.3)}{100} \times \frac{280}{(1.1.4)} \times \frac{(1.1.5)}{760} \times 1000 \\ &= 3.68 \frac{(1.1.3) \times (1.1.5)}{(1.1.4)} \end{aligned}$$

1.2 Radioactive Noble Gases

- 1.2.1 Decay time in minutes _____
- 1.2.2 Sample count time in seconds _____
- 1.2.3 Detector _____
- 1.2.4 Geometry _____
- 1.2.5 Common multiplier* _____
- 1.2.6 Volume of sidearm flask _____

$$\begin{aligned} \text{*CM} &= \frac{280}{(1.1.4)} \times \frac{(1.2.6)}{0.5} \\ &= 5.6 \times 10^2 \frac{(1.2.6)}{(1.1.4)} \end{aligned}$$

(1) See Section 5.2.2 of EPIP 7.3.2.

1.2.7 MCA Results

<u>Isotope</u>	<u>Concentration (μCi/cc)</u>
Xe-133	_____
Kr-85m	_____
Kr-88	_____
Xe-133m	_____
Xe-135	_____
Xe-138	_____
Kr-87	_____
Xe-135m	_____
Ar-41	_____
Kr-85	_____
Xe-131m	_____

2.0 ANALYSIS OF LIQUID SAMPLE2.1 Boron and pH

2.1.1	Volume of sample	_____ ml
2.1.2	pH of sample	_____
2.1.3	Normality of NaOH	_____
2.1.4	Volume of NaOH used	_____ ml
2.1.5	Concentration of boron*	_____ ppm

$$*ppm \text{ Boron} = \frac{(2.1.5)}{(2.1.1)} \times (2.1.4) \times 10810$$

2.2 Chloride

2.2.1	Chloride concentration	_____ pp
-------	------------------------	----------

2.3 Iodine Analysis

2.3.1	Decay time	_____ min
2.3.2	Sample count time	_____ sec
2.3.3	Detector	_____
2.3.4	Geometry	_____

2.3.5 Common multiplier* _____

$$*CM = \frac{1075}{V_s} \times F$$

V_s = Volume of sample (first dilution)

F = Any additional dilution factors (2 or more dilutions)

(1) See Section 5.4.3 of EPIP 7.3.2.

2.3.6 MCA Results

<u>Isotope</u>	<u>Concentration ($\mu\text{Ci/cc}$)</u>
I-130	_____
I-131	_____
I-132	_____
I-133	_____
I-134	_____
I-135	_____
Total	_____

3.0 ROUTING AND APPROVAL

	<u>Date</u>	<u>Initials</u>
3.1 Calculations completed by	_____	_____
3.2 Analysis completed by	_____	_____
3.3 Chemistry & Health Physics Supervisor	_____	_____
3.4 Chemistry/Health Physics Supervisor (TSC)	_____	_____
3.5 Director (TSC)	_____	_____

POST-ACCIDENT SAMPLING OF CONTAINMENT ATMOSPHERE

1.0 INTRODUCTION

This procedure outlines the steps necessary to collect, handle and analyze a potentially high radioactive containment atmosphere sample resulting from gross fuel failure and loss of reactor coolant system integrity to determine hydrogen and radioactive gas concentrations.

2.0 PRELIMINARY EVALUATION

NOTE: THE FOLLOWING EVALUATION SHALL BE COMPLETED PRIOR TO ANY ATTEMPT TO ENTER THE FACADE OR R11/R12 CUBICLE TO PERFORM A VALVE LINEUP OR COLLECT A CONTAINMENT ATMOSPHERE SAMPLE.

2.1 Indications of Possible Fuel Damage

Some or all of the following would be present if fuel damage or loss of reactor coolant system integrity had occurred:

- 2.1.1 The letdown radiation monitor (R9) and failed fuel monitor (RE-109) would be unusually high or offscale.
- 2.1.2 The containment radiation monitors 1R11 and 1R12, or 2RE-211 and 2RE-212 would be unusually high or offscale.
- 2.1.3 The containment area monitors 1R2 and 1R7 or 2RE-102 and 2RE-107 would be unusually high or offscale.
- 2.1.4 The automatic actions of EOP-1A have caused containment isolation.

2.2 Evaluation of Radiological Hazards in Access Areas Required for Sampling

Initials

- 2.2.1 After evaluation of the radiation monitoring system readouts, verify with Health Physics that the appropriate airborne and radiation surveys have been made before entering the facade.

Initials

2.2.2 Verify the requirements for facade and 1R11/1R12 or 2RE-211/2RE-212 cubicle entry.

- a. Radiation work permit requirements
- b. Clothing requirements
- c. Respiratory requirements
- d. Dosimetry requirements including extremity dose monitoring
- e. Health physics coverage requirements including timekeeping

3.0 CONTAINMENT ATMOSPHERE SAMPLING PROCEDURE

NOTE: THE FOLLOWING PROCEDURE SHALL NOT BE INITIATED UNTIL THE EVALUATION DISCUSSED IN SECTION 2.0 HAS BEEN COMPLETED. THE DUTY & CALL SUPERINTENDENT (COORDINATOR), THE DUTY AND CALL HEALTH PHYSICS SUPERVISOR AND THE DUTY SHIFT SUPERINTENDENT SHALL APPROVE THE IMPLEMENTATION OF THIS PROCEDURE. THE FOLLOWING STEPS WILL BE ACCOMPLISHED UNDER THE DIRECTION OF HEALTH PHYSICS SUPERVISION.

3.1 Unit 1 Containment Atmosphere Sampling Using the 1R11/1R12 Sampling System

3.1.1 Valve Lineup

- a. Entry into the facade shall be from the potable water room.
- b. Verify with the control room that the 1R11/1R12 sampling system is lined up in accordance with the following:
 - (1) The containment isolation valves 3200C and 3200G are closed.
 - (2) The 3200A&B AOV's in the R11/R12 cubicle are closed.
 - (3) The 1P707 forced purge pump is secured.

Initials

c. Proceed to the R11/R12 cubicle, and place the 1R11/1R12 sampling system in the following lineup:

(1) Open valves 1-3200J and 1-3200L. _____

(2) Close valve 1-3200M. _____

(3) Verify that the continuous vent valves 1-3200S and 1-3200W are closed. _____

(4) Close valve 1-3200K. _____

d. Connect the 100' service air hose to the Chicago fitting located on the elevator side of the cubicle established for sample line purging. _____

NOTE: THE AIR HOSE AND PRESSURE REGULATOR ARE STORED INSIDE THE POTABLE WATER ROOM.

e. Verify that valve 1-3200Y is closed. _____

f. Proceed to the potable water room with the air hose and pressure regulator.

NOTE: FOR RCT TRAINING EXERCISES, DO NOT USE THE POTABLE WATER ROOM SERVICE AIR HOOKUPS, SINCE THEIR USE UNNECESSARILY BURDENS SECURITY. THE SERVICE AIR HOOKUPS LOCATED ON THE EL. 66' CONTROLLED SIDE FACADE WALL ARE TO BE USED.

(1) Attach the service air hose to an available turbine hall service air outlet using the pressure regulator designated for this procedure. _____

(2) Lock the Chicago fitting in place using the appropriate pins. _____

g. Verify with the control room that the 1R11/1R12 monitor is lined up to monitor, and discharge to, containment atmosphere. _____

h. Request the control room to open the 1-3200A&B AOV sample system valves and the containment isolation valves 1-3200C&G. _____

Initials

- i. Verify sample flow by an increase in radiation levels in the cubicle, and return to the potable water room. _____
- j. Allow five minutes for sample recircing. Request control room to stop the pump and close 1-3200A, B, C & G AOV isolation valves. _____

3.1.2 Sample Collection

- a. Enter the facade and proceed to the sample point. Take along two 2 cc gas syringes in a hollowed-out lead break. One syringe should have 1 cc of AgZ inside the syringe barrel. _____

CAUTION: CONTAINMENT PRESSURE COULD THEORETICALLY BE AS HIGH AS 60 PSIG. HOLD THE SYRINGE PLUNGER SECURELY. ALSO, LOCK THE GAS SYRINGES BEFORE WITHDRAWING FROM THE SAMPLING SEPTUM, USING THE BUILT-IN LOCKING DEVICE ON THE SYRINGE.

- b. Remove the set screw and take two 1 cc gas samples. Place the syringes in the lead brick. _____
- c. Replace the set screw and open valve 3200Y. _____
- d. Leave the facade and return to the potable water room. _____

3.1.3 Sample Line Purging

- a. Open service air valve and adjust the pressure to a setting of 10 psig greater than measured containment pressure. _____
- b. Request control room to open the pump discharge isolation valve 1-3200A. _____
- c. Request control room to start the 1R11/1R12 pump. _____
- d. Verify purge effectiveness by measuring the reduction in radiation levels after approximately 15 minutes of purging. _____

Initials

- e. Request control room to close the 1-3200A pump discharge isolation valve, secure the pump, and open the 1-3200B&C pump suction isolation valves. _____
- f. Continue the purge for 10 more minutes. _____
- g. Request control room to close all pump discharge and suction isolation valves and secure the pump. _____
- h. Secure the service air valve. _____

NOTE: AFTER DRILLS AND PRACTICE RUNS,
RETURN ALL EQUIPMENT AND VALVE
LINEUPS TO THE AS-FOUND CONDITION.

3.2 Unit 2 Containment Atmosphere Sampling Using the RE-211/RE-212
Sampling System, After Containment Isolation

3.2.1 Valve Line-Up

- a. Verify with the control room that the following containment isolation conditions exist:
 - (1) Valves 3200A, B & C have closed. _____
 - (2) Pumps P707A&B have been secured. _____
 - (3) Valve 3200G is lined up to return flow to the containment (Position 1). _____
 - (4) Valves 3200D, E, F & H are closed. _____
- b. Obtain the designated 100' service air line and pressure regulator from the potable water room. _____
- c. Proceed to the nonnuclear room with the air hose and pressure regulator. _____

NOTE: FOR RCT TRAINING EXERCISES, DO NOT
USE THE NONNUCLEAR ROOM SERVICE AIR
HOOKUPS, SINCE THEIR USE UNNECESSARILY
BURDENS SECURITY. THE SERVICE AIR
HOOKUPS LOCATED ON EL. 66' CONTROLLED
SIDE FACADE WALL ARE TO BE USED.

Initials

- (1) Attach the service air hose to an available turbine hall service air outlet using the pressure regulator designated for this procedure. _____
- (2) Lock the Chicago fitting in place, using the appropriate pins. _____
- d. Proceed to the RE-211/RE-212 cubicle and perform the following:
 - (1) Connect the service air hose to the Chicago fitting attached to valve 3200Y, and verify that the valve is closed. _____
 - NOTE: THE VALVE IS LOCATED ON THE ELEVATOR SIDE OF THE R211/R212 CUBICLE.
 - (2) Lock the Chicago fitting in place, using the appropriate pins. _____
 - (3) Enter the cubicle and perform the following:
 - (1) Verify AOV-3200A&B are closed by visual observation. _____
 - (2) Manually close valves 3200K, N, P, Q, W, EE & FF. _____
 - (3) Manually open valves 3200J & L. _____

- e. Request the control room perform the following:

NOTE: THE SAMPLE PUMP P707A SEALS ARE RATED FOR 5 PSI. TO PREVENT AN INADVERTENT AIRBORNE RELEASE, THE CONTAINMENT PRESSURE MUST BE LESS THAN 5 PSI BEFORE PROCEEDING.

- (1) Verify containment pressure is less than 5 psi. _____
- (2) Place the sample mode switch located in the cable spreading room in the SEPTUM position. _____

Initials

(3) Open AOV's 3200A&B and the containment isolation valve 3200C.

(4) Start sample pump P707A.

f. Verify sample flow by observing a flow increase on FIT-3288 and return to the nonnuclear room.

g. Allow 15 minutes for sample recirculation and return to the cubicle with two (2) 2 cc gas-tight syringes in a hollowed-out lead brick. One syringe should have 1 cc of AgZ inside the syringe barrel.

3.2.2 Sample Collection

a. Obtain two 1 cc gas samples from the sample septum by means of the gas-tight syringes.

NOTE: LOCK THE SYRING PLUNGERS PRIOR TO WITHDRAWING FROM THE SEPTUM.

b. Place the syringes in the leak bricks for transport.

c. Establish a service air pressure of 5 psi on the regulator and open valve 3200Y.

3.2.3 Sample Line Purge

a. Return to the nonnuclear room and request the control room close valves 3200B&C.

b. Allow the system to purge forward for 25 minutes, then request the control room perform the following:

(1) Secure the sample pump P707A.

(2) Close AOV-3200A and place valve 3200M in its normal position (Position 1).

(3) Open AOV-3200B&C.

c. Allow the system to purge backwards 25 minutes and request the control room close valves 3200B&C.

Initials

- d. Return to the cubicle and perform the following:
- (1) Verify the effectiveness of the purge by measuring the reduction of the radiation levels. _____
 - (2) Close the service air hose regulator, close valve 3200Y and remove the service air hose. _____
 - (3) Enter the cubicle and open valves 3200K, N, P & Q. _____
 - (4) Close valves 3200J & L. _____
- e. Notify the control room that sampling has been completed, and that valves 3200W, EE & FF are closed. _____

NOTE: AFTER ALL DRILLS AND PRACTICE EXERCISES, ALL EQUIPMENT AND VALVE LINEUPS ARE TO BE RETURNED TO THE AS-FOUND CONDITION.

4.0 CONTAINMENT ATMOSPHERE ANALYSIS

4.1 Volume Adjustments

- 4.1.1 Before proceeding with the hydrogen, radioactive noble gas and radioactive iodine analyses, the sample contents of both syringes must be brought to atmospheric pressure. Use the shielded sacrificial glass bomb for this purpose.
- 4.1.2 Insert the syringe through the rubber septum of the glass bomb. Unlock the syringe locking device and let the syringe and bomb equilibrate for approximately 30 seconds. Relock the syringe and withdraw from the septum. Store the syringe in the lead brick.
- 4.1.3 Repeat Step 4.1.2 for the other syringe.

4.2 Hydrogen

Assuming normal equipment setup and preparations are complete, use the sample injection port on the gas partitioner, inject a 1 cc gas sample and proceed using normal hydrogen analysis procedures. _____

Initials

4.3 Radioactive Noble Gases and Iodines

Noble Gases

- 4.3.1 Use the 2 cc syringe with the 1 cc of AgZ for the noble gas and iodine separation/analysis.
- 4.3.2 Inject the 1 cc containment atmosphere sample into a one liter poly bottle. Remove the syringe and store in the lead brick. Tape over the syringe hole in the poly bottle. Let the gas come to equilibrium for a minimum of 15 minutes while proceeding with the iodine analysis section.
- 4.3.3 Withdraw a 1 cc gas sample from the poly bottle in Step 4.3.2. If the contact reading on the syringe is less than 1 mR/hr, inject the sample into a 5 cc vial and count as normal. If the contact reading is greater than 1 mR/hr, further dilutions are necessary.

Dilution procedure: Inject the 1 cc sample into a one liter poly bottle and allow to equilibrate. Withdraw 1 cc from the poly bottle. If the contact reading is less than one mR/hr, inject into a 5 cc vial and count as normal. If the contact reading is greater than 1 mR/hr, dilute further.

NOTE: FOR EACH DILUTION, USE A NEW SYRINGE OR MAKE CERTAIN THE OLD SYRINGE IS COMPLETELY PURGED. LABEL NOBLE GAS SAMPLES AND IODINE SAMPLES WITH ALL PERTINENT INFORMATION SUCH AS: SAMPLE NUMBER, NAME OF SAMPLE, DATE AND TIME OF SAMPLE, SAMPLE VOLUME AND DILUTION(S).

Iodines

- 4.3.4 Unscrew the syringe tip and internal plunger stop and pour the silver zeolite into a counting test tube and cap.
- 4.3.5 If the silver zeolite/iodine sample is less than one mR/hr contact, count the sample directly on the MCA using the test tube geometry.

- 4.3.6 If the iodine activity is too high to count directly, determine the radiation level in mR/hr at one foot. Also, determine the percent isotopic composition using the MCA and attenuating the sample if necessary.

Determine the total iodine activity using the following equation:

$$C_{(Ci)} = \frac{R/hr @ 1'}{6E_T}$$

The following table can be used as an aid in determining the "Total Average Energy/ γ " (E_T) for the above equation.

Isotope	(1) Average γ Energy	(2) % Composition (Fraction)	(1) x (2) Weighted γ Energy
I ¹³¹	0.380	_____	_____
I ¹³²	0.731	_____	_____
I ¹³³	0.530	_____	_____
I ¹³⁴	0.857	_____	_____
I ¹³⁵	1.238	_____	_____

Total Weighted γ Energy : (E_T) _____

Each individual iodine isotopic concentration ($\mu\text{Ci/cc}$) is calculated as follows:

$$\mu\text{Ci/cc } I_{(i)} = C_{(Ci)} \times 2 \times 10^6 \times (\text{Fractional } \% \text{ Composition})$$

5.0 CALCULATIONS

5.1 Pressure and Temperature Correction

Ask control room supervision for the temperature and pressure in containment (psig and $^{\circ}\text{F}$). Determine atmospheric pressure (mm Hg) and temperature ($^{\circ}\text{F}$) from the barometer in the laboratory. Convert this to psia. Apply the following correction factor to all results:

$$\text{Concentration } (\mu\text{Ci/cc}) \times \frac{(P + 14.7)}{P_{\text{lab}}} \times \frac{T}{T_{\text{lab}} + 459} \quad \text{Initials} \quad \underline{\hspace{2cm}}$$

$$P_{\text{lab}} (\text{psia}) = 14.7 \text{ psi} \times \frac{P (\text{mm Hg})}{760 \text{ mm Hg}}$$

- 5.2 Complete and forward Containment Atmospheric Post-Accident Sampling Analysis Report (EPIP-31).

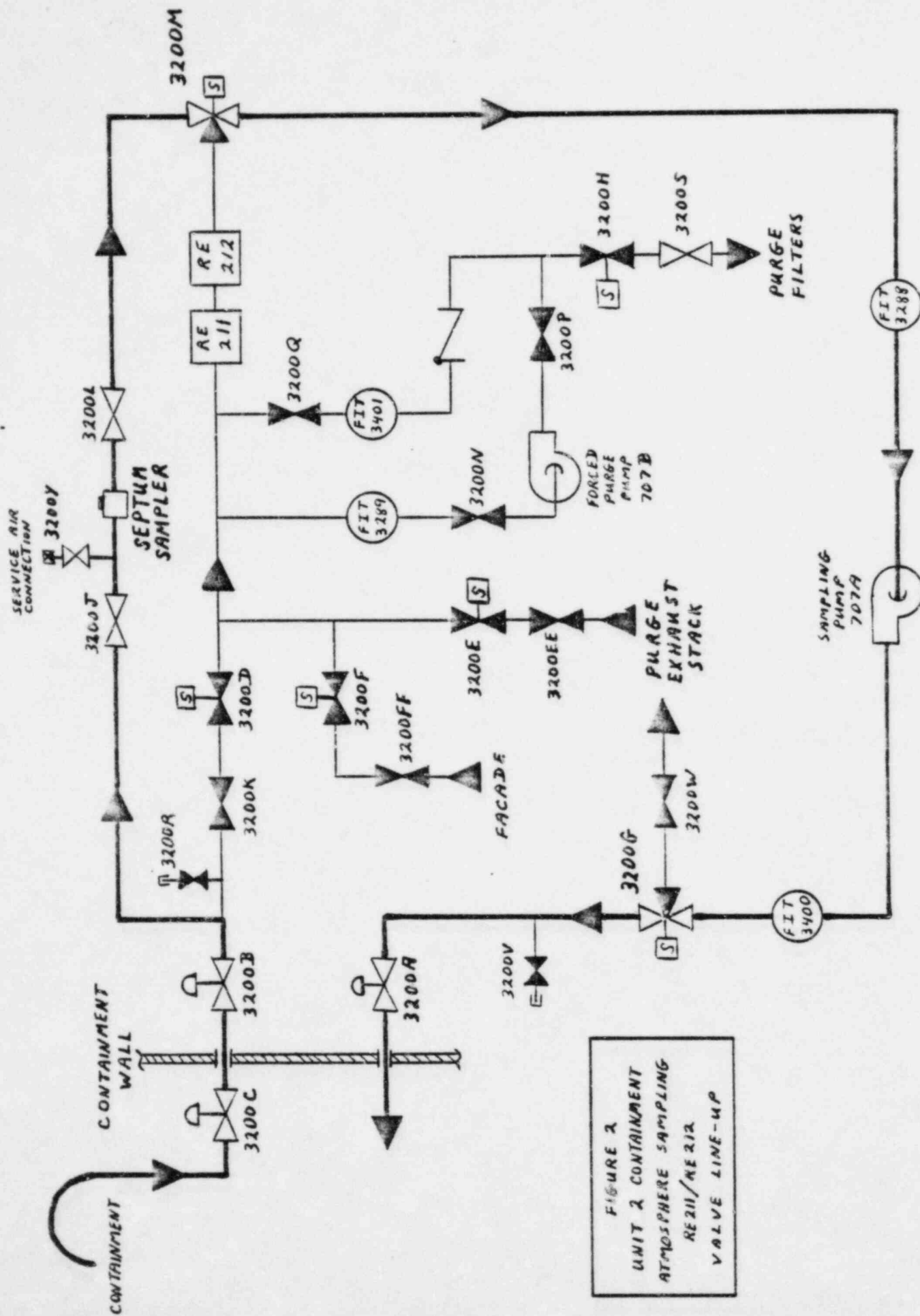


FIGURE 2
UNIT 2 CONTAINMENT
ATMOSPHERE SAMPLING
VALVE LINE-UP

POINT BEACH NUCLEAR PLANT

CONTAINMENT ATMOSPHERE POST-ACCIDENT SAMPLING ANALYSIS REPORT

1.0 HYDROGEN ANALYSIS

- 1.1 Scale used on gas partitioner _____
- 1.2 Millivolt reading on chart recorder _____
- 1.3 % hydrogen from calibration curve _____

2.0 RADIOACTIVE NOBLE GAS ANALYSIS

- 2.1 Decay time in minutes _____
- 2.2 Sample count time in seconds _____
- 2.3 Detector _____
- 2.4 Geometry _____
- 2.5 Number of dilutions : (D) _____
- 2.6 Common multiplier* _____

$$*CM = (D)^1 \times \frac{1075}{1.0} \times (\text{Pressure-Temperature Correction})$$

$$\text{Pressure-Temperature Correction} = \frac{(P + 14.7)(T + 459)}{P_L (T_L + 459)}$$

Where P_C = Pressure in Containment (psig) _____

T_C = Temperature in Containment ($^{\circ}F$) _____

P_L = Pressure in Lab (psi) where _____

$$P_L = 14.7 \times \frac{P \text{ (mm Hg)}}{760 \text{ (mm Hg)}}$$

T_L = Temperature in Laboratory ($^{\circ}F$) _____

- (1) See Section 4.3.3 of EPIP 7.3.3. One or more dilutions may be necessary.

2.7 Results of MCA

<u>Isotope</u>	<u>Concentration ($\mu\text{Ci/cc}$)</u>
Xe-133	_____
Kr-85m	_____
Kr-88	_____
Xe-133m	_____
Xe-135	_____
Xe-138	_____
Kr-87	_____
Xe-135m	_____
Ar-41	_____
Kr-85	_____
Xe-131m	_____

3.0 RADIOACTIVE IODINE ANALYSIS

- 3.1 Decay time in minutes _____
- 3.2 Sample count time in seconds _____
- 3.3 Detector _____
- 3.4 Geometry _____
- 3.5 Results of MCA

<u>Isotope</u>	<u>Concentration ($\mu\text{Ci/cc}$)</u>
I-131	_____
I-132	_____
I-133	_____
I-134	_____
I-135	_____

4.0 ROUTINE & APPROVAL

	<u>Date</u>	<u>Initials</u>
4.1 Calculations completed by	_____	_____
4.2 Analysis completed by	_____	_____
4.3 Chemistry & Health Physics Supervisor	_____	_____
4.4 Chemistry/Health Physics Supervisor (TSC)	_____	_____
4.5 Director (TSC)	_____	_____