

NRC FORM 366
(12-81)
10 CFR 50U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORTAPPROVED BY OMS
3159-0011CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 A L B R F 2 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

CONT

01 REPORT SOURCE L 8 0 5 0 0 0 2 6 0 7 0 7 2 0 8 3 6 0 8 1 8 8 3 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During transient operation following a control rod sequence exchange, R calcu-

03 lated (FRP/CMFLPD) was less than R setpoint (.947 < .949) for two hours 11-

04 minutes. Technical Specification 2.1.A.1 requires R calculated to be greater

05 than or equal to R setpoint. There was no effect on public safety or health.

06 The APRM high neutron flux trip was operable at all times during the event.

07

08

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE
Z Z 11 X 12 Z 13 Z 7 Z Z Z Z 14 Z 15 Z 16

17 LER/RO REPORT NUMBER 8 3 0 4 3 0 3 L 0

18 ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-6 FORM SUB PRIME COMP SUPPLIER COMPONENT MANUFACTURER
X 18 Z 19 Z 20 Z 21 0 0 0 Y 23 N 24 Z 25 Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The root cause of the event was a xenon transient in conjunction with operation

11 closer to thermal limits due to the core loading required because of fuel

12 failures in cycle 4. Control rods were repositioned to restore the core within

13 limits. Tech. Spec. changes have been submitted to allow 6 hours to correct R.

14

15 FACILITY STATUS N POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)
E 28 0 7 3 29 NA A 31 Engineer Observed

16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (38)
Z 33 Z 34 NA NA

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)
0 0 0 37 Z 38 NA

18 PERSONNEL INJURIES NUMBER DESCRIPTION (41)
0 0 0 40 NA

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)
Z 42 NA

20 PUBLICITY ISSUED DESCRIPTION (45) NA

NAME OF PREPARED Bill T. Williamson

PHONE (205) 729-0845

8308260149 830818
PDR ADOCK 05000260
S PDR

NRC USE ONLY

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

83 AUG 23 A 9:47

August 18, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE
REPORT BFRO-50-260/83043

The enclosed report provides details concerning a calculated R value that
was less than the required setpoint value. This report is submitted in
accordance with Browns Ferry unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

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Washington, D.C. 20555

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NRC Inspector, Browns Ferry

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IE 22

LER SUPPLEMENTAL INFORMATION

BFRO-50-260 / 83043 Technical Specification Involved 2.1.A.1

Reported Under Technical Specification 6.7.2.B.2 * Date Due NRC 8/19/83

Event Narrative:

Unit was in refuel outage; unit 3 was operating at 99-percent power. Neither unit was affected by this event. Unit 2 was operating at 73-percent under transient conditions following a control rod sequence exchange. During normal core monitoring, it was observed R calculated (FRP/CMFLPD) was less than R setpoint (.947 < .949.) Technical Specification 2.1.A.1 requires R calculated to be greater than or equal to R setpoint. TIP traverses were performed to verify the calculation.

The root cause of the event was a xenon transient combined with operation closer to thermal limits due to core loading required because of fuel failures in cycle 4. Control rods were repositioned to restore R to within limits in two hours 11-minutes. Technical Specification Submittal 167 was submitted in October 1981, to allow 6 hours for correction of R factor problems. This is similar to technical specifications already in effect for unit 1.

There was no effect on public health and safety. The APRM high neutron flux trips were available and operable at all times during this event.

* Previous Similar Events:

BFRO-50-259/80053, 80056, 80078
260/81005, 81006, 81007, 83033, 83035, 83039
296/79003, 81018

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRR