

3150-0011

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'TEVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE						COMP SUBCODE		VALVE SUBCODE	
0	9	Z	Z	A		B		Z	Z	Z	Z	Z	Z	Z	
17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
LEAK/NO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE						REPORT TYPE		REVISION NO.	
8	3					0	4	1		/	0	3	L		0
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPD-4 FORM SUB.		PRIME COMP. SUPPLIER	
H	Z	Z		Z		Z		0	0	0	0	Y	N	Z	
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48
COMPONENT MANUFACTURER															
Z	9	9	9												

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
1	5	E	20	0	9	8	29	NA	30	A	31	Schedule checks	32

PERSONNEL EXPOSURES									
NUMBER		TYPE		DESCRIPTION					
1	7	0	0	0	37	2	38	NA	

8 9 11 12				NA
LOSS OF OR DAMAGE TO FACILITY				
TYPE		DESCRIPTION		

ISSUED		DESCRIPTION		(45)	
0	N	44			
8	8	10			

8308260130 830817
PDR ADDCK 05000296
S PDR

NRC USE ONLY

PHONE (205) 729-0861

NRC USE ONLY

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 83041 Technical Specification Involved 4.3.A.2

Reported Under Technical Specification 6.7.2.b.(2) * Date Due NRC 8/18/83

Event Narrative:

Unit 1 was in a refueling outage; Unit 2 was operating at approximately 96% power. These units were not affected by this event. Unit 3 was the only unit affected by this event and was operating at approximately 98% power when the event was identified. Technical Specification 4.3.A.2 requires each fully or partially withdrawn control rod to be exercised one notch each week when above 30% power. The failure to perform Surveillance Instruction (SI) 4.3.A.2 within the normal surveillance band of July 16, 1983 to July 18, 1983 was identified by operations SI scheduling personnel at approximately 1300 hours on July 19, 1983. Unit 3 was immediately placed in a six (6) hour limiting condition of operation per Technical Specification definition 1.0.C.1 and the SI was completed at 1320 hours on July 19, 1983. Failure to perform SI 4.3.A.2 until approximately 13-1/2 hours outside the normal surveillance band prevented the possible identification of a potential inoperable control rod until 13-1/2 hours outside the normal SI band. There was no effect on public health and safety. SI 4.3.A.2 identified no inoperable control rods and the Standby Liquid Control System was operable during the entire event as a backup means of reactivity control. Failure to run the SI was traced to the fact that the SI was not placed on the operator's Rolodex file of scheduled SI's due to it being missed in the proofing process when the operations SI's were transferred from the SI schedule to the operator's Rolodex file. Operators who performed this proofing have been informed of the importance of accurate proofing of required SI scheduling.

* Previous Similar Events:

BFRO-50-259/80009, 82042, 82051, 83027
260/81010
296/82015

The above-listed licensee event reports relate to surveillance instructions on various systems not performed during the schedule band.

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

USNRC
ATLANTA
TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

August 17, 1983

83 AUG 23 9:48

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFRO-50-296/83041

The enclosed report provides details concerning a control rod exercise
surveillance instruction not performed within the required time band.
This report is submitted in accordance with Browns Ferry unit 3 Technical
Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. J. Green
H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

OFFICIAL COPY

IE 2211