

USNRC REGION II
ATLANTA, GEORGIA
TELECOPY TO NRC
Aug. 17, 1983

83 AUG 23 A 9:46

U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, NW
Suite 3100
Atlanta, Georgia 30303

BFRO-50-259/83049R5

Reported under Technical Specification 6.7.2.a.(3)

Telecopy Date 8/17/83 Time 1500

Date of Occurrence: 8/12/83 Time of Occurrence: 1400 Unit 1

Technical Specification Involved: 3.6.G

Conditions Prior to Occurrence

Unit 1 in refueling outage
Unit 2 at 3139 MWt
Unit 3 at 3081 MWt

Identification and Description of Occurrence

Weld DCS-1-7 (core spray) has an intermittent indication on the pipe side of the weld that is 360-degrees in length and approximately 41% through wall depth. This is a 12-inch, 304 stainless steel, schedule 80 pipe. The cracking is between the manual isolation valve and the testable check valve (inboard containment isolation valve).

Weld DSRWC-1-3 (reactor water cleanup) has two indications that are approximately 5" long between 11:30 and 2:30 and 1-1/2" long between 8:30 and 9:30 with a through-wall depth of greater than 80%. This is a 6-inch, 304 stainless steel, schedule 304 pipe. The cracking is between the inboard and outboard containment isolation valves.

Weld DCS-1-8 (core spray) has two indications that are approximately 9" long between 12:30 and 3:30 and 3" long between 9:30 and 10:30 with a through-wall depth of approximately 30%. This is a 12-inch, 304 stainless steel, schedule 80 pipe. The cracking is on the reactor vessel side of the pipe-to-valve weld on valve HCV-75-27 (manual isolation valve).

Weld DSRWC-1-2 (reactor water cleanup) has a indication that is approximately 3" long between 12:00 and 1:00 with a through-wall depth of greater than 50%. This is a 6-inch, 304 stainless steel, schedule 80 pipe. The cracking is between the inboard and outboard containment isolation valves.

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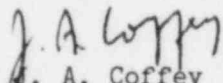
Corrective Action

Because of the additional weld cracks discovered, an additional 29 welds will be inspected. This will increase the total number of welds to be inspected from 53 to 82 (see attachment for the number of welds per system and the inspection status). This includes all stainless steel welds greater than 1" diameter in all safety-related systems. These 82 welds are composed of Class 1 and 2 stainless steel and dissimilar metal (carbon steel to stainless steel) circumferential butt welds that fall within the scope of Section XI. This will consist of:

- . Class 1, stainless steel welds on the CS System, RWCU System and on the RHR head spray line.
- . Class 1 and 2, dissimilar metal (carbon steel to stainless steel) welds.
- . Class 2, stainless steel portion of the HPCI System, CS System and RHR discharge piping.

Update Item

To date the attached are our inspection results.


J. A. Coffey
Acting Power Plant Superintendent
Browns Ferry Nuclear Plant

ATTACHMENT - BFRO 50-259/83049R5

| | <u>Total Inspection Required</u> | <u>Inspection Complete</u> | <u>Inspection Satisfactory</u> | <u>Welds with Cracks</u> | <u>Under Evaluation</u> |
|------------------|--------------------------------------|--------------------------------|------------------------------------|------------------------------|-----------------------------|
| HPCI | 5 | 0 | 0 | 0 | 0 |
| Core Spray | 34 | 25 | 23 | 2 | 0 |
| RWCU | 14 | 4 | 2 | 2 | 0 |
| RHR (Class 1) | 1 | 0 | 0 | 0 | 0 |
| RHR (Head Spray) | 27 | 0 | 0 | 0 | 0 |
| RHR (Discharge) | 2 | 0 | 0 | 0 | 0 |
| CRD | 1 | 0 | 0 | 0 | 0 |
| TOTAL | 84 | 44 | 40 | 4 | 0 |

SATISFACTORY

| | |
|------------------|--|
| Core Spray | DCS 1-6, 9, 14, 15, 18, 16, 17, 5, 4 DSCS 1-4, 5, 6, 8, 9, 10, 11, 12, 13, 3, 1, 2, 7, 13 |
| RHR (Head Spray) | DHS 1-2, 3 DSHS 1-1A, 3, 4, 5, 6, 7, 8, 9, 10, 11, 12, 13, 14 |
| RWCU | DRWC 1-4 DSRWC 1-7 |

UNDER EVALUATION

None

This is an update of the event first reported previously on 8/11/83, and includes all the results through 1330 on 8/17/83. This information was provided verbally to Ross Butcher on 8/17/83.

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BFR0-50-259/83049R4

Reported under Technical Specification 6.7.2.a.(3)

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Technical Specification Involved: 3.6.G

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Weld DSRWC-1-2 (reactor water cleanup) has a indication that is approximately 3" long between 12:00 and 1:00 with a through-wall depth of greater than 50%. This is a 6-inch, 304 stainless steel, schedule 80 pipe. The cracking is between the inboard and outboard containment isolation valves.

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Corrective Action

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J. A. Coffey

Acting Power Plant Superintendent
Browns Ferry Nuclear Plant

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|------------------|--------------------------------------|--------------------------------|------------------------------------|------------------------------|-----------------------------|
| HPCI | 5 | 0 | 0 | 0 | 0 |
| Core Spray | 34 | 24 | 20 | 2 | 2 |
| RWCU | 14 | 3 | 1 | 2 | 0 |
| RHR (Head Spray) | 27 | 15 | 15 | 0 | 0 |
| RHR (Discharge) | 2 | 0 | 0 | 0 | 0 |

SATISFACTORY

| | |
|------------------|--|
| Core Spray | DCS 1-6, 9, 14, 15, 18, 16, 17, 5 LSCS 1-4, 5, 6, 8, 9, 10, 11, 12, 13, 3, 1, 2 |
| RHR (Head Spray) | DHS 1-2, 3 DSHS 1-1A, 3, 4, 5, 6, 7, 8, 9, 10, 11, 12, 13, 14 |
| RWCU | DRWC 1-4 |

UNDER EVALUATION

| | |
|------------------|---------------------|
| Core Spray | DSCS 1-7 DCS 1-4 |
| RWCU | None |
| RHR (Head Spray) | None |

This is an update of the event first reported previously on 8/11/83, and includes all the results through 1330 on 8/16/83. This information was provided verbally to Floyd Cantrell on 8/16.