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609 971-4000
Writer's Direct Dial Number:

October 12, 1994
C321-94-2158

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Dear Sir:

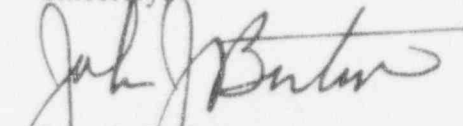
Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report 94-016

This letter forwards one copy of Licensee Event Report 94-016.

This LER is the result of a Local Leak Rate Test (LLRT) failure. The LLRT activity will continue as scheduled. Upon completion of the testing and evaluation of the results, a revised LER will be submitted.

If there are any questions please contact Mr. John Rogers of my staff at 609.971.4643.

Sincerely,



John J. Barton
Vice President and Director
Oyster Creek

JJB/JR
Enclosure

cc: Administrator, Region I
Senior Resident Inspector
Oyster Creek NRC Project Manager

9410170221 941012
PDR ADDCK 05000219
S PDR

GPU Nuclear Corporation is a subsidiary of General Public Utilities Corporation

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NRC FORM 366M (5-92)						U.S. NUCLEAR REGULATORY COMMISSION APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95																													
LICENSEE EVENT REPORT (LER)																																			
FACILITY NAME (1) <div style="text-align: center;">Oyster Creek, Unit 1</div>						DOCKET NUMBER (2) <div style="text-align: center;">05000219</div>			PAGE (3) <div style="text-align: center;">1 OF 3</div>																										
TITLE (4) <div style="text-align: center;">Local Leak Rate Test Results In Excess of Technical Specification Limits</div>																																			
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER																									
09	12	94	94	016	0	10	12	94	FACILITY NAME	DOCKET NUMBER																									
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)																																	
POWER LEVEL (10)		<table border="0" style="width: 100%;"> <tr> <td style="width: 25%; padding: 5px;">20.402(b)</td> <td style="width: 25%; padding: 5px;">20.405(c)</td> <td style="width: 25%; padding: 5px;">50.73(a)(2)(iv)</td> <td style="width: 25%; padding: 5px;">73.71(b)</td> </tr> <tr> <td style="padding: 5px;">20.405(a)(1)(i)</td> <td style="padding: 5px;">50.36(c)(1)</td> <td style="padding: 5px;">50.73(a)(2)(v)</td> <td style="padding: 5px;">73.71(c)</td> </tr> <tr> <td style="padding: 5px;">20.405(a)(1)(ii)</td> <td style="padding: 5px;">50.36(c)(2)</td> <td style="padding: 5px;">50.73(a)(2)(vii)</td> <td style="padding: 5px;">OTHER</td> </tr> <tr> <td style="padding: 5px;">20.405(a)(1)(iii)</td> <td style="padding: 5px; text-align: center;">X</td> <td style="padding: 5px;">50.73(a)(2)(i)</td> <td style="padding: 5px;">50.73(a)(2)(viii)(A)</td> </tr> <tr> <td style="padding: 5px;">20.405(a)(1)(iv)</td> <td></td> <td style="padding: 5px;">50.73(a)(2)(ii)</td> <td style="padding: 5px;">50.73(a)(2)(viii)(B)</td> </tr> <tr> <td style="padding: 5px;">20.405(a)(1)(v)</td> <td></td> <td style="padding: 5px;">50.73(a)(2)(iii)</td> <td style="padding: 5px;">50.73(a)(2)(x)</td> </tr> </table>										20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER	20.405(a)(1)(iii)	X	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	20.405(a)(1)(iv)		50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	20.405(a)(1)(v)		50.73(a)(2)(iii)	50.73(a)(2)(x)
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LICENSEE CONTACT FOR THIS LER (12)																																			
NAME Terry Bitowf						TELEPHONE NUMBER (Include Area Code) 609-971-4649																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS																									
SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)			MONTH	DAY	YEAR																								
X YES (If yes, complete EXPECTED SUBMISSION DATE).						NO			04	30	95																								
ABSTRACT (Limit to 1400 spaces i.e. approximately fifteen single spaced lines) (16)																																			
<p>On September 12, 1994, Local Leak Rate Testing (LLRT) results indicated that Main Steam Isolation Valve NS04B exceeded the leak rate limit of 12.08 SCFH at 20 psig as specified in the plant Technical Specifications.</p> <p>The leak rate was quantified as 36.91 SCFH at 35 psig. The LLRT program for the current outage has not been completed at this time. Any additional LLRT failures, and a complete evaluation of the program results, will be reported after the program has been completed this outage.</p>																																			

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Oyster Creek, Unit 1	05000219	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
		94	-- 016 --	0	

DATE OF OCCURRENCE

The condition being reported was discovered on 09/12/94.

IDENTIFICATION OF OCCURRENCE

Main Steam Isolation Valve (MSIV) NS04B (EIIIS SB-ISV) exceeded the leak rate criteria specified in Technical Specification 4.5.F.2.

This condition is reportable under 10 CFR 50.73(a)(2)(i).

CONDITIONS PRIOR TO OCCURRENCE

The Plant was in a cold shutdown condition for a refueling outage when this condition was discovered.

DESCRIPTION OF OCCURRENCE

On September 12, 1994, Local Leak Rate Testing (LLRT) results indicated that Main Steam Isolation Valve NS04B exceeded the leak rate limit of 12.08 SCFH at 20 psig as specified in plant Technical Specifications. The leak rate was quantified as 36.91 SCFH at 35 psig.

APPARENT CAUSE OF OCCURRENCE

The cause of this occurrence has not been determined at this time and will be reported in a revision to this LER.

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Oyster Creek, Unit 1	05000219	94	-- 016 --	0	3 OF 3

ANALYSIS OF OCCURRENCE AND SAFETY ASSESSMENT

The safety significance of this event has not been determined at this time and will be reported in a revision to this LER.

CORRECTIVE ACTION

Corrective action will be dependant on the root cause determination and cannot be specified at this time. The corrective actions taken for the LLRT program in the 15R outage will be included in the revision to this LER.

SIMILAR EVENTS

LER 82-014 LLRT Failure

LER 82-019 LLRT Results on MSIVs Outside Limits

LER 82-020 LLRT Results on MSIV's Greater than Limits

LER 83-025 Containment Penetration Found Degraded Due to Isolation Valves Actuator/Valve Linkages Out of Adjustment

LER 91-002 Local Leak Rate Test Results in Excess of Limits Due to Valve Degradation

LER 92-013 Local Leak Rate Test Results in Excess of Limits Due to Valve Degradation