

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK: 

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	S	C	H	B	R	2	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5
7	8	9						14	15	25										26	30					57	58		
		LICENSEE CODE								LICENSE NUMBER											LICENSE TYPE								

CON'T

REPORT SOURCE	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80				
	L	6	0	5	0	0	0	2	6	1	7	0	8	0	3	8	3	8	0	8	1	7	8	3	9
	DOCKET NUMBER									EVENT DATE					REPORT DATE										

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On August 3, 1983, with the unit at 79% power, the current Technical Specification cooldown curves were determined to be in error. This determination was made following a review of new draft curves which are based on the analysis of a reactor vessel surveillance capsule removed during the 1982 Refueling Outage. This event is reported pursuant to Technical Specification 6.9.2.a.8. H. B. Robinson, Unit 2 Operating Procedures do not permit operation in the area of the curves that are of concern and, therefore, this event does not represent a threat to the public health and safety.

09		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22
		Z	Z	X		X		Z	Z	Z	Z	Z	Z	Z	Z
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.					
23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38
		8	3		0	2	0	/	0	1	T		0		
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER	
39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54
G	G	Z	Z	Z				0	0	0	0	Y	N	N	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER	
55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The operating curves were revised on August 4, 1983, to include the conservative  
1 1 aspects of both the new draft curves and the existing curves. These composite curves  
1 2 will be used until the new curves are finalized by the NSSS Vendor and included in  
1 3 the Plant Technical Specifications. No further corrective action is considered  
1 4 necessary.

8 9  
FACILITY STATUS  
1 5 E 28  
% POWER  
0 7 9 29  
OTHER STATUS 30 N/A  
METHOD OF DISCOVERY  
C 31  
DISCOVERY DESCRIPTION 32 Special Review

ACTIVITY CONTENT  
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35)  
1 6 4 33 2 34 N/A  
2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)	N/A	

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
1	8	0	0
0	0	0	40
		N/A	

		8		9		10		11		12		
				TYPE		DESCRIPTION		(43)				
1	9	Z		(42)		N/A						

2 0 8 9 10  
 PUBLICITY  
 ISSUED DESCRIPTION (45)  
 N 44  
 8308260061 830817  
 PDR ADOCK 05000261  
 S PDR  
 NRC USE ONLY  
 68 69 70

NAME OF PREPARER Howard T. Cox

68 69

PHONE: (803) 383-4524

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SUPPLEMENTAL INFORMATION  
FOR  
LICENSEE EVENT REPORT 83-020

I. Cause Description and Analysis

On August 3, 1983, with the unit at 79% power, it was determined that the current Technical Specification cooldown curves were in error.

During the 1982 Refueling Outage, a Reactor Vessel Material Surveillance Capsule (Capsule T) was removed for analysis. Based on the results of that analysis, new heatup and cooldown curves were requested from the NSSS Vendor. These new curves were to cover a period from 10 to 20 effective full power years (EFPY) and were expected to yield higher cooldown rate curves compared to existing curves. The existing curves were prepared by a different vendor based on data obtained from a previously removed surveillance capsule (Capsule V).

On August 3, 1983, following review of the new draft curves, it was determined that at temperatures less than 210°F and 160°F for cooldown rates of 100°F per hour and 60°F per hour respectively, the draft curves (10 EFPY) are less conservative than the existing TS cooldown curves (20 EFPY). At 100°F the 100°F per hour cooldown curve is 70 psig less conservative and the 60°F per hour cooldown curve is 20 psig less conservative. The existing cooldown curves are based on thermal conductivities taken from Section III of the 1977 ASME Boiler & Pressure Vessel Code. These thermal conductivity values were corrected in the 1980 edition of the ASME Code, and therefore, the existing cooldown curves based on the previous ASME values are in error. Conversations with the vendor who produced the existing curves (Southwest Research Inc.) indicated that at the time they identified the change in the ASME Code they understood that CP&L was in the process of revising the Robinson curves with another vendor. They, therefore, felt that it was unnecessary to notify CP&L of the error. They had indicated, however, that others affected by this error were notified.

This event is reported pursuant to Technical Specification 6.9.2.a.8. The H. B. Robinson, Unit 2 Operating Procedures do not permit operation in the less conservative area of the curve as described above. Therefore, this event does not represent a threat to the public health and safety.

II. Corrective Action

The operating cooldown curves were revised on August 4, 1983, to include the conservative aspects of both the new draft curves and the existing curves. These composite curves will be used until the new curves are finalized by the vendor and included in the plant Technical Specifications.

III. Corrective Action to Prevent Recurrence

No further corrective action is considered necessary.



Carolina Power & Light Company

62 AUG 23 4 9:37  
H. B. ROBINSON STEAM ELECTRIC PLANT  
Post Office Box 790  
Hartsville, South Carolina 29550

AUG 17 1983

Robinson File No: 13510C

Serial: RSEP/83-1081

Mr. James P. O'Reilly  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261  
LICENSE NO. DPR-23  
LICENSEE EVENT REPORT 83-020

Dear Mr. O'Reilly:

In accordance with Section 6.9.2 of the Technical Specifications for the H. B. Robinson Steam Electric Plant, Unit 2, the enclosed Licensee Event Report is submitted. This report fulfills the requirements for a written report within fourteen (14) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July, 1977.

Very truly yours,

R. B. Starkey, Jr.  
General Manager  
H. B. Robinson SEG Plant

HTC:FMG:FLL:CWC:JMC/th

Enclosure

cc: R. C. DeYoung (40)  
R. A. Hartfield (3)

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