

## LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)  
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/7/2/9/8/3/ (8) /0/8/1/7/8/3/ (9)  
 SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On July 29, 1983, with Unit 1 at 100 percent reactor power, the Individual Rod /  
 /0/3/ / Position Indication (IRPI) for rod J-13 indicated 110 steps when it had previous- /  
 /0/4/ / ly, and should have read 228 steps. The IRPI for rod J-13 was repaired, recal- /  
 /0/5/ / brated and returned to service within 1 hour therefore the health and safety of /  
 /0/6/ / the public were not affected. This event is contrary to T.S. 3.1.3.2 and report- /  
 /0/7/ / able pursuant to T.S. 6.9.1.9.b. /  
 /0/8/ /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/	/I/D/ (11)	/E/ (12)	/E/ (13)	/I/N/S/T/R/U/ (14)	/X/ (15)
	LER/RO	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE
(17)	REPORT NUMBER				REVISION NO.

/0/9/	/I/D/ (11)	/E/ (12)	/E/ (13)	/I/N/S/T/R/U/ (14)	/X/ (15)	/Z/ (16)
	LER/RO	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.
(17)	REPORT NUMBER					
	/8/3/	/-/	/0/5/2/	/-/	/0/3/	/L/
	/-/	/0/				

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

/B/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /N/ (24) /N/ (25) /W/1/2/0/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Erroneous rod position indication was caused by a bad connection of the signal /  
 /1/1/ / conditioning module for IRPI of rod J-13. The connection was reseated and the /  
 /1/2/ / IRPI recalibrated satisfactorily. Incore detectors were used to verify rod /  
 /1/3/ / position during the event. No further corrective action is required. /  
 /1/4/ /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/	/E/ (28)	/1/0/0/ (29)	/ NA / (30)	/A/ (31) / Operator Observation /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/	/Z/ (33)	/Z/ (34)	/ NA /

PERSONNEL EXPOSURES	NUMBER	TYPE	DESCRIPTION (39)
/1/7/	/0/0/0/ (37)	/Z/ (38)	/ NA /

PERSONNEL INJURIES	NUMBER	DESCRIPTION (41)
/1/8/	/0/0/0/ (40)	/ NA /

LOSS OF OR DAMAGE TO FACILITY	TYPE	DESCRIPTION (43)
/1/9/	/Z/ (42)	/ NA /

PUBLICITY ISSUED	DESCRIPTION (45)	NRC USE ONLY
/2/0/	/N/ (44)	/ NA /

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

**Vepco**

USNRC REGION II  
ATLANTA, GEORGIA

83 AUG 23 4:37  
VIRGINIA ELECTRIC AND POWER COMPANY  
NORTH ANNA POWER STATION  
P. O. BOX 402  
MINERAL, VIRGINIA 23117

August 17, 1983

Mr. James P. O'Reilly, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 2900  
Atlanta, Georgia 30303

Serial No. N-83-118  
NO/CLF: 11  
Docket No. 50-328  
License No. NPF-4

Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

Report No.

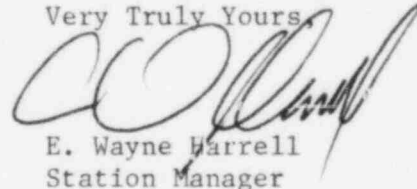
Applicable Technical Specifications

LER 83-052/03L-0

T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,



E. Wayne Harrell  
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)  
016 Phillips Bldg.  
U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

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