Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 16
BF 15.2

JUL 27 1983

USNRC REGION II
ATLANTA, GEORGIA

LER TELEPHONE AND TELECOPY FORMAT

U. S. Nuclear Regulatory Commission
Region II
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83 AUG 23 A 9:46

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BF(E)RO-50- 259 / 83049

Reported under Technical Specifications 6.7.2. a(3)

Telecopy: Date _____ Time _____

Telephone: Date 8/11/83 Time _____

Person Contacted Ross Butcher

Date of Occurrence 8/11/83 Time of Occurrence _____ Unit 1

Technical Specification involved: 3.6.G

Conditions Prior to Occurrence:

Unit 1 - Refueling Outage
Unit 2 - 3139 MWt
Unit 3 - 3081 MWt

Identification and Description of Occurrence:

Additional ultrasonic inspections of piping welds (on the reactor system pressure boundary headspray, core spray, and reactor water cleanup systems,) revealed indications of cracking. Cracking was confirmed on two welds; one on the core spray system and the other on the reactor water cleanup system. Weld DSRWC-1-3 (Reactor Water Cleanup) has two indications that are approximately 5" long

(Continuation on next page)

Apparent Cause of Occurrence:

Cracks appear to be intergranular stress corrosion cracking.

Other Related Events:

BFRO 50-260/82040
259/83023

Corrective Action Taken or Planned:

Additional welds are being inspected and details of inspection findings and corrective actions will be submitted periodically.

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

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*Revision

IE22
1/1

Ross Butcher

BFRO-50-259/830409

Identification and Description of Occurrence: (Continued)

between 11:30 and 2:30, and 1-1/2" long between 8:30 and 9:30 with a through wall depth of greater than 80-percent. This is a 6-inch, 304 stainless steel, schedule 80 pipe. The cracking is between the inboard and outboard automatic isolation valves.

Weld DCS-1-7 (core spray) has an intermittent indication on the pipe side of the weld that is 360-degrees in length and approximately 41-percent through wall depth. This is a 12-inch, 304 stainless steel, schedule 80 pipe. The cracking is between the reactor vessel and the manual isolation valve.

Issuance of this prompt report confirms discussions with NRC, Region II on this date.