

CONTROL BLOCK (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 ALBRF1 200-000000-000341111145
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60CONT
01 REPORT SOURCE L 605000259707208380818839
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 While performing SI 4.7.E.5 (CREV Flowrate Test), it was found that control room
03 emergency ventilation system (CREV) B failed to achieve the flowrate required by
04 T.S. 3.7.E.2.c of 500 cfm $\pm 10\%$. This system is common to units 1, 2 and 3.
05 The CREV system would supply control room air in an emergency situation involv-
06 ing contamination of normal plant ventilation. There was no effect on
07 public health or safety. Redundant system (CREV Unit A) was available and
08 operable.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
S G 11 E 12 B 13 VALVE X 14 L 15 G 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

17 LER/RO REPORT NUMBER 83
21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-4 FORM SUB PRIME COMP. SUPPLIER COMPONENT MANUFACTURER (26)
X 18 Z 19 Z 20 Z 21 00000 Y 23 N 24 L 25 X 9 9 9 9
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The flow is set by positioning a manual damper. The wing nut that held the
11 damper at its preset position had vibrated loose and allowed the damper to
12 partially close. The flowrate was properly reset and a lock nut installed on
13 the damper. This is an isolated event and does not require recurrence control.

14

15 FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)
H 28 000 29 NA B 31 Surveillance Testing
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
Z 33 Z 34 NA NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)
000 37 Z 38 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

18 PERSONNEL INJURIES NUMBER DESCRIPTION (41)
000 40 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)
Z 42 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

20 PUBLICITY ISSUED DESCRIPTION (45)
N 44 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

NAME OF PREPARER J. O. Ely

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8308260004 830818
PDR ADOCK 05000259
S PDR

NRC USE ONLY

LER SUPPLEMENTAL INFORMATION

BFRO-50-259 / 83041 Technical Specification Involved 3.7.E.2.c

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 8/19/83

Event Narrative:

Unit 1 was in refuel outage, unit 2 at 2752 MW(th) and unit 3 at 3225 MW(th). During performance of SI 4.7.E.5 (Control Room Emergency Ventilation (CREV) System Flowrate Test), unit B was found to have a flowrate less than 500 cfm ± 10 -percent as required by Technical Specification (T.S.) 3.7.E.2.c. This system is common to units 1, 2, and 3. A Limiting Condition of Operation (LCO) of seven days continued operation was entered upon event discovery. The CREV system would supply control room air in an emergency situation involving contamination of normal plant ventilation. There was no danger to the health or safety of the public. A redundant system (CREVS "A") was available and operable.

The flow is set by positioning a manual damper. The manual discharge damper wing nut had vibrated loose and allowed the damper to partially close. The flow was immediately reset to that required and a lock nut was installed on the damper. This is considered a random failure and no recurrence control is needed. A lock nut was also installed on CREVS "A" manual discharge damper.

* Previous Similar Events:

NRC Inspection Report 83-09 (Violation D)

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

REGION II
TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

83 AUG 23

9:48

August 18, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303


Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE
REPORT BFRO-50-259/83041

The enclosed report provides details concerning failure of the control room
emergency ventilation system to achieve the required flowrate of
500 CFM + 10 percent. This report is submitted in accordance with Browns
Ferry unit 1 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY


H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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