10 CFR 50.73

Virginia Electric and Power Company
North Anna Power Station
P. O. Box 402
Mineral, Virginia 23117

October 11, 1994

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

NAPS: MPW
Docket No. 50-338
License No. NPF-4

Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Unit 1.

Report No. 50-338/94-006-00

This Report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,

J. A. Stall
Station Manager

Enclosure:

cc: U.S. Nuclear Regulatory Commission
101 Marietta Street, N.W.
Suite 2900
Atlanta, Georgia 30323

R. D. McWhorter
NRC Senior Resident Inspector
North Anna Power Station

IE22
1/1

NRC FORM 366 (5-92) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, DC 20503.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
North Anna Power Station Unit 1	05000338	1 OF 3

TITLE (4)
PRESSURIZER SAFETY VALVES OUTSIDE TECHNICAL SPECIFICATION LIMITS DUE TO SETPOINT DRIFT

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
										05000
09	17	94	94	006	00	10	11	94	FACILITY NAME	DOCKET NUMBER 05000

OPERATING MODE (9)	6
POWER LEVEL (10)	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)

	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(B)
	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(C)
	20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER
	20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iv)		50.73(a)(2)(i)		50.73(a)(2)(viii)(B)		
	20.405(a)(1)(v)		50.73(a)(2)(i)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Mr. J. A. Stall	(703) 894-2101

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	AB	RV	D243	Y						

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	YES (If yes, completed EXPECTED SUBMISSION DATE)	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On September 17, 1994, with Unit 1 in Mode 6 (Refueling), the "as found" set pressures for one Pressurizer Safety Valve was found to be outside the setpoint tolerances allowed by Technical Specifications 3.4.3. On September 18, 1994 a second Pressurizer Safety Valve was also found to be outside the setpoint tolerances allowed by Technical Specifications. These events are reportable pursuant to 10CFR50.73 (a)(2)(i)(B) for conditions prohibited by Technical Specifications.

Three pressurizer safety valves and fifteen main steam safety valves were sent to Wyle Labs for testing to ensure conformance to Technical Specifications requirements. The "as found" set pressures for two Pressurizer Safety Valves were found to be outside Technical Specification tolerance of plus or minus 1 percent of the setpoint pressure of 2485. The safety valves were repaired and readjusted within the correct setpoint tolerance allowed by Technical Specifications.

This event posed no significant safety implications because the safety valves would have performed their safety function in the event of an overpressure condition. The health and safety of the public were not affected at any time during this event.

1.0 Description of the Event

On September 17, 1994, with Unit 1 in Mode 6 (Refueling), the "as found" set pressures for one Pressurizer Safety Valve (EIIS System AB, Component RV, Vendor Identifier D243) (1-RC-SV-1551B) was found to be outside the setpoint tolerances allowed by Technical Specifications 3.4.3. On September 18, 1994 a second Pressurizer Safety Valve (1-RC-SV-1551C) was also found to be outside the setpoint tolerances allowed by Technical Specifications.

Three pressurizer safety valves (PSV) and fifteen main steam safety valves were sent to Wyle Labs for testing to ensure conformance to Technical Specifications requirements. Testing was performed in accordance with Periodic Test procedures 1-PT-50, "Pressurizer Code Safety Valve Setpoint Verification" and 1-PT-70, "Main Steam Code Safety Valve Setpoint Verification, respectively. The "as found" set pressures for two Pressurizer Safety Valves were found to be outside Technical Specification tolerance of 2485 psig plus or minus 1 percent. The safety valves were repaired and readjusted within the correct setpoint tolerance allowed by Technical Specifications.

2.0 Significant Safety Consequences and Implications

This event posed no significant consequences or safety implications because the "as found" set pressure settings for the PSVs are bounded by the safety analyses for over pressure transients and the acceptance criteria for overpressure accidents would have been met. Therefore, the health and safety of the public were not affected at any time during this event.

These events are reportable pursuant to 10CFR50.73 (a)(2)(i)(B) for conditions prohibited by Technical Specifications.

3.0 Cause of the Event

The cause of the event has been determined to be setpoint drift. The industry has experienced a history of setpoint drift for safety valves of this type and is a common occurrence.

4.0 Immediate Corrective Actions

The PSVs were refurbished and retested at Wyle Laboratories to within the allowable limits of Technical Specification 3.4.3.

5.0 Additional Corrective Actions

No additional actions are required.

6.0 Actions to Prevent Recurrence

Westinghouse Owners Group WCAP-12910 Revision 1-A, Pressurizer Safety Valve Set Pressure Shift, was issued in May 1993. The NRC has reviewed the WCAP and supplemental information and determined the assessment of PSV set pressure shift and PSV response to plant over pressure conditions is adequately conservative and acceptable. The guidance provided by the WCAP is used when establishing the PSV setpoints.

7.0 Similar Events

Previous similar events where Pressurizer (PZR) Safety Valves and/or MSSVs have been outside the requirements of Technical Specification 3.4.3 have occurred at North Anna Power Station on Unit 1 on March 2, 1981 (LER 81-040/03L-0), May 6, 1987 (LER 87-008-00), February 5, 1992 (LER 92-002-00), January 20, 1993 (LER N1-93-002-00) and on Unit 2 on March 23, 1982 (LER 82-014/03L-0), September 13, 1987 (LER 87-008-00), April 12, 1989 (LER 89-006-00), October 17, 1990 (LER 90-005-00), and March 31, 1992 (LER 92-003-00).

8.0 Additional Information

Unit 2 was operating at 100 percent power (mode 1) and was not affected by this event.

The following are the setpoint and as found pressures:

Pressurizer Safety Valves	Set Pressure (PSIG)	As Found (PSIG)
1-RC-SV-1551B	2485 +/- 25	2576
1-RC-SV-1551C	2485 +/- 25	2518

NAPS LER N1-94-006-0

I. VERIFICATION OF ACCURACY

1. Station Deviation Reports N94-1313 dated September 17, 1994.
2. Station Deviation Reports N94-1327 dated September 18, 1994.
3. Pressurizer & Main Steam Safety Valve Status Report dated September 21, 1994.
4. Nuclear Analysis and Fuel Report ET No. NAF-94077, Revision 0, dated October 5, 1994, As Found Lift Setpoints Pressurizer Safety Valves.
5. Westinghouse Owner's Group WCAP-12910, Report on Pressurizer Safety Valve Setpoint Shift, Revision 1-A.
6. NRC Safety Evaluation of Topical Report WCAP 12910 dated February 19, 1993.

II. ACTION PLAN

1. Pressurizer Safety Valves were refurbished and retested at Wyle Laboratories to within the allowable limits of Technical Specification 3.4.3.

III. COMMITMENTS (STATED OR IMPLIED)

None