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ARTHUR E. LUNDVALL, JR.  
VICE PRESIDENT  
SUPPLY

August 22, 1983

Director of Nuclear Reactor Regulation  
Attention: Mr. R. A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: Calvert Cliffs Nuclear Power Plant  
Units Nos. 1 & 2; Dockets Nos. 50-317 and 50-318  
Unit 1 Seventh Cycle License Application and  
Request for Unit 1 & 2 License Amendment

Gentlemen:

The Baltimore Gas and Electric Company hereby requests an amendment to its Operating License No. DPR-53 for Calvert Cliffs Unit 1 to allow operation for a seventh cycle. The Company also requests amendments to its Operating Licenses Nos. DPR-53 and DRP-69 to reflect planned modifications to the auxiliary feedwater system and the remote shutdown panel. These modifications are scheduled to be completed during the upcoming refueling outage at Unit 1.

Pursuant to the procedural requirements of 10CFR50 paragraphs 50.91 and 50.92 this letter forwards our advance notification of the proposed license amendments and provides a preliminary determination with regard to significant hazards considerations. This application will be supplemented no later than September 6, 1983 with a detailed description of the required technical specification changes and with supporting safety analysis information.

Our present intention is to begin the Unit 1 refueling outage on October 1, 1983 and to complete the outage and begin the approach to criticality on November 23, 1983 with a return to power operations immediately thereafter.

#### Proposed Changes

The required technical specification changes associated with the core reload will be identified in the supplement mentioned above. The changes will be similar in nature to those requested for previous core reload license applications to the extent that the new fuel assemblies to be loaded into the Cycle 7 core will be of the same design as those used previously, the acceptance criteria in the technical specifications will not significantly differ from that contained in the existing technical specifications, and the analytical methods used to evaluate the core reload have not been changed.

The Unit 1 auxiliary feedwater system (AFWS) is being modified to include a third train consisting of a new motor-driven pump and associated controls powered from the emergency diesel generator safety bus. In addition, a cross connect between the Unit 1 and Unit 2 motor-driven AFWS trains will be placed into operation to maximize the reliability benefits provided by the existence of these trains.

In support of these physical changes we are proposing various changes to the action statement and surveillance requirements of Technical Specification 3/4.7.1.2 for Units 1 and 2. We are also proposing an expansion of the discussion in the basis for Technical Specification 3.7.1.2 (p. B 3/4 7-2) to explicitly identify the FSAR accident analyses that are potentially affected by AFWS performance.

Finally, several technical specification changes are being proposed in association with planned changes to our newly-installed alternate safe shutdown panel. This panel is located on Elevation 45' of the safety-related switchgear room (immediately adjacent to the Control Room) and replaces the existing remote shutdown panel (1C43) located in the steam-driven auxiliary feedwater pump room. The new alternate safe shutdown panel, and the changes to the panel proposed by this letter, are made necessary by the fire protection requirements of 10CFR50.48 and 10CFR50, Appendix R. Affected are Technical Specification Tables 3.3-9, 3.3-10, 4.3-6, and 4.3-10.

Detailed descriptions of the technical specification changes supporting the AFWS and alternate safe shutdown panel modifications will be provided in the forthcoming supplement to this letter.

#### Determination of Significant Hazards Considerations

We have made a preliminary determination based on analytical information presently available that this Amendment request does not involve a significant hazards consideration. This conclusion was derived by applying the Commission guidance for implementation of 10CFR50.92. Specifically, no fuel assemblies to be loaded into the Cycle 7 core will be of new or different design than those used previously and found to be acceptable to the NRC. No proposed changes to the Technical Specifications for Cycle 7 involve acceptance criteria that are significantly different than those previously found acceptable to the NRC. The analytical methods used to demonstrate performance with the technical specifications and regulations are consistent with those previously approved by the NRC.

To evaluate the impact of the AFWS modifications on future operations and postulated plant upset conditions, we have reevaluated those transients that are affected by AFWS performance; i.e., the main steam line break, feedwater line break, and loss of feedwater flow events. Based upon the preliminary results of these transient analyses, we conclude that the proposed AFWS technical specification changes do not involve a significant hazards consideration, in that:

- (1) The probability or consequences of these events are not significantly increased,
- (2) Operation with the modified AFWS under the proposed technical specifications does not create the possibility of a new or different kind of accident from any accident previously evaluated; and
- (3) The changes do not involve a significant reduction in a margin of safety as described in the technical specification bases.

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Detailed safety analysis information in support of the above changes will be submitted in the forthcoming supplement to this letter following review and approval by the Plant Operations and Safety Review Committee (POSRC) and the Off-Site Safety Review Committee (OSSRC).

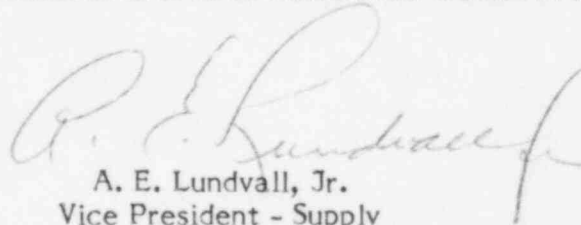
The proposed technical specification changes associated with the alternate safe shutdown panel are minor in nature, are related to modifications that have been approved by the NRC under the Appendix R fire protection review, and as such, do not involve a significant hazards consideration.

#### Fee Determination

We have determined, pursuant to 10CFR Part 170 paragraph 170.22, that this Amendment request, with forthcoming supplements, consists of a Class IV and a Class I Amendment and accordingly we are including BG&E Check Number A114518 in the amount of \$12,700.00 to cover the fee for this request.

BALTIMORE GAS AND ELECTRIC COMPANY

BY:

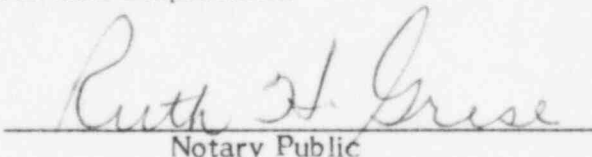
  
A. E. Lundvall, Jr.  
Vice President - Supply

STATE OF MARYLAND :  
:  
CITY OF BALTIMORE :

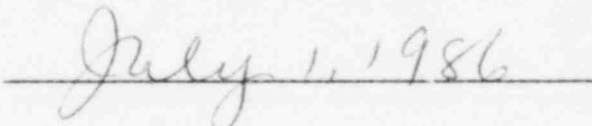
TO WIT:

Mr. A. E. Lundvall, Jr., being duly sworn, states that he is Vice President of the Baltimore Gas and Electric Company, a corporation of the State of Maryland; that he executed the foregoing for the purposes therein set forth; that the statements made are true and correct to the best of his knowledge, information, and belief; and that he was authorized to execute the same on behalf of said Corporation.

Witness my Hand and Notarial Seal:

  
Notary Public

My Commission Expires:

  
July 1, 1986

AEL/BSM/vf

cc: Messrs. J. A. Biddison, Jr., Esq.  
G. F. Trowbridge, Esq.  
Mr. D. H. Jaffe, NRC  
Mr. R. E. Architzel, NRC  
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