



Log # TXX-91296  
File # 10110  
CP-86-41  
Ref. # 10CFR50.55(e)

William J. Cahill, Jr.  
Executive Vice President

August 13, 1991

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SMALL BREAK LOCA MODE 4 OPERATION  
SDAR CP-86-41 (INTERIM REPORT)

REF: 1) TU Electric Letter logged TXX-90288 from William J. Cahill, Jr.  
to NRC dated August 13, 1990

Gentlemen:

On May 28, 1986, TU Electric orally notified your Mr. I. Barnes of a deficiency involving evaluation of the effect of a small break Loss of Coolant Accident (LOCA) during Mode 4 operation. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

As of the last interim report (reference 1), the Westinghouse Owner's Group (WOG) had not issued a final report. Specific issues associated with the switch over of the Residual Heat Removal System to the safety injection mode had not been resolved.

TU Electric has continued to participate in WOG activities concerning the Shutdown LOCA Analysis. A draft report has been issued by the WOG Analysis Subcommittee for review and comment by WOG members. The results of analysis described in the draft report confirm that the current Emergency Core Cooling System design, as well as the operator action time, are adequate for meeting the requirements of 10 CFR 50.46 for a LOCA during Mode 3 and Mode 4 operation.

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The WOG final report is expected to be issued in the Fall of 1991. Based on the final report, TU Electric will provide a status update on this issue by February 27, 1992.

Sincerely,

William J. Cahill, Jr.

By: J. S. Marshall  
J. S. Marshall  
Generic Licensing Manager

DNB/dnb

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (?)  
Mr. T. A. Bergman, NRR  
Mr. M. B. Fields, NRR