



Commonwealth Edison

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August 16, 1983

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: LaSalle County Station, Units 1 & 2
Thermal Adequacy of Suppression Pool
(In-Plant SRV Test Results)
NRC Docket Nos. 50-373 and 50-374

Dear Mr. Denton:

Commonwealth Edison Company provided confirmation of the LaSalle containment adequacy per NUREG 0808 criteria via DAR Appendix I submittal on September 24, 1982. Subsequent to that report, in December 1982, an in-plant SRV discharge test was run to validate the thermal adequacy of the suppression pool. Among other objectives from that test was the verification of the suppression pool temperature monitoring system (SPTMS) to measure and indicate the thermal response of the pool during and following SRV blowdown events. LaSalle SER 6.2 (NUREG 0519) requested that the bulk-to-local pool temperature differences be established by the LaSalle in-plant SRV test and that the maximum local pool temperature be verified.

Prior suppression pool local and bulk temperature limits were based on NUREG-0487 analyses. A Tech Spec local temperature limit of 200°F was established from that basis. However, by resolution of NRC Task Action Plan 39, this limit was revised in NUREG-0783 to be dependent upon steam mass flux and the amount of subcooling of suppression pool water near the steam quench front. For LaSalle station this temperature bound ranges between 200°F and 216.5°F for local temperatures based on NUREG-0783.

The actual suppression pool thermal response to SRV discharge was measured as reported in the attached S&L report "LaSalle County 1 In-plant S/RV Test, Evaluation of Suppression Pool Temperature Measurements". Results show the mean local-to-bulk pool temperature difference to be 8.1°F (the 95/95 confidence level for non exceedance is 12°F). Test results also confirm that the SPTMS provides a conservative measure of bulk pool temperature. An evaluation of the LaSalle Technical Specifications relative to the currently imposed analytical pool limits was also made; the present specifications were found to have a 20°F margin for suppression pool response to SRV blowdown.

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August 16, 1983

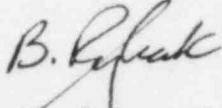
This report fulfills the SER 6.2 requirement for actual test data on LaSalle suppression pool thermal response to SRV discharge phenomena.

To the best of my knowledge and belief the statements contained herein and in the Enclosure are true and correct. In some respects these statements are not based on my personal knowledge but upon information furnished by other Commonwealth Edison and contractor employees. Such information has been reviewed in accordance with Company practice and I believe it to be reliable.

Enclosed for you use please find one signed original and forty (40) copies of this report and the enclosure.

If there are any questions concerning this matter, please contact this office.

Very truly yours,


for C. W. Schroeder
Nuclear Licensing Administrator

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cc: NRC Resident Inspector - LSCS

Enclosure

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