

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK: 1 2 3 4 5

0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CON'T
0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
On 7/9/83 at 0830 hrs. during LIS-RI-01, RCIC Steam Line High Flow Isolation Calib.
& Functional Test, DPIS 1E31-N013B low dp setpoint was found to be out of tolerance
per Tech. Spec. 3.3.2. DPIS 1E31-N013A setpoints were found within the Tech. Spec.
allowable value. This switch would have isolated RCIC Outboard Steam Line Isolation
Valve 1E51-F008, had isolation been required. DPIS 1E31-N013B would have functioned
properly at a high value than the Tech. Spec.

0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
The cause for the setpoint drift in 1E31-N013B could not be determined. All switch
internals functioned properly. The switch is ITT Barton, Model 288A. The setpoints
were immediately recalibrated per LIS-RI-01 on 7/9/83. Trend analysis may assist in
further corrective action on these instruments.

0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

NAME OF PREPARER R. W. Houston

PHONE (815) 357-6761

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- I. LER NUMBER: 83-076/03L-0
- II. LASALLE COUNTY STATION: Unit 1
- III. DOCKET NUMBER: 050-373
- IV. EVENT DESCRIPTION:

On July 9, 1983, at 0830 hours during the performance of LIS-RI-01, RCIC Steam Line High Flow Isolation Calibration and Functional Test, Differential Pressure Indicating Switch 1E31-N013B low dP setpoint was found to be out of tolerance in the non-conservative direction. Technical Specification 3.3.2 Allowable Value of less than or equal to 185" H₂O was exceeded by 37" H₂O.

V. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

At the time of the surveillance the reactor was in the RUN Mode producing 2716 MWT, 734 MWE. DPIS 1E31-N013A setpoints were found to be within the Allowable Value. This switch would have isolated the RCIC Outboard Steam Line Isolation Valve, 1E51-F008, had an isolation been required. DPIS 1E31-N013B would have functioned properly, at a steam line flow higher than the Tech. Spec. Allowable Value. Safe plant operation was maintained.

Other problems associated with the 1E31-N013 instruments were reported in LER #82-091 and LER #83-014.

VI. CAUSE:

The cause for the setpoint drift in 1E31-N013B could not be determined. All switch internals functioned properly. This switch was manufactured by ITT Barton, Model 288A.

VII. CORRECTIVE ACTION:

The setpoint of 1E31-N013B was immediately recalibrated per the LIS-RI-01 procedure on 7/9/83.

Trend analysis of LIS-RI-01 per LAP 100-12 may assist in further corrective action on these instruments.

Prepared by: R. W. Houston



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LaSalle County Nuclear Station
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August 2, 1983

James G. Keppler
Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Sir:

Reportable Occurrence Report #83-076/03L-0 Docket #050-373 is being submitted to your office in accordance with LaSalle County Nuclear Power Station Technical Specification 6.6.B.2.(a). Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/GW/rg

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U.S. NRC Document Management Branch
INPO-Records Center
File/NRC

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