

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONT

0	1
---	---

REPORT SOURCE

L	6	0	5	0	0	0	2	8	0	7	0	7	1	8	8	3	8	0	8	1	5	8	3	9
---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

7 8

0	9
---	---

9 10 11 12 13 14 15 16 17 18 19 20

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE		
W	A	E		B		V	A	L	V	E	X	C		0

(17) LER/RO REPORT NUMBER [EVENT YEAR 8 3] [21 22] [23] [24] [0 3 2] [26] [27] [28] [0 3] [29] [30] [L] [31] [32] [0]
 ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
 [X] [18] [Z] [19] [33 34] [Z] [20] [35] [Z] [21] [36] [0 0 0 0] [37 40] [Y] [23] [41] [Y] [24] [42] [A] [25] [43] [P 3 0 5] [26] [44 47]

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

7		8		9		FACILITY STATUS						% POWER				OTHER STATUS (30)				METHOD OF DISCOVERY				DISCOVERY DESCRIPTION (32)												80	
1		5		E		(28)		1		0		0		(29)		N/A				B		(31)		Operator Observation												80	
7		8		9				10		11		12		13		44				45		46														80	

PERSONNEL EXPOSURES										
NUMBER			TYPE	DESCRIPTION (39)						
1	7	0	0	0	37	Z	38	N/A		

8	9	11	12	8308220303 830815	
LOSS OF OR DAMAGE TO FACILITY				(43)	
TYPE		DESCRIPTION		PDR	ADOCK 05000280
1	9	Z	(42)	S	PDR
				N/A	

IE22
11

7	8	9	10											80	
			PUBLICITY	(45)											NRC USE ONLY
ISSUED			DESCRIPTION	(44)											
2	0	N	N/A												
7	8	9	10											68 69 80	

NRC USE ONLY

PHONE: (804) 357-3184

ATTACHMENT 1
SURRY POWER STATION, UNIT NO. 1
DOCKET NO: 50-280
REPORT NO: 83-032/03L-0
EVENT DATE: 07-18-83

TITLE OF THE EVENT: 1-SW-P-10B LOW DISCHARGE PRESSURE

1. Description of the Event

On July 18, 1983, with the unit at 100% power, service water pump 1-SW-P-10B would not develop sufficient discharge pressure. Upon investigation, it was determined that the discharge check valve, 1-SW-113, for pump 1-SW-P-10A was not properly seated. This event is contrary to T.S.3.3.A.8.b and is reportable per T.S. 6.6.2.b.(2).

2. Probable Consequences and Status of Redundant Equipment

With the 10A pump check improperly seated, discharge from the 10B pump can backflow through the 10A pump, thereby reducing the service water flow to the charging pump coolers.

The redundant charging pump service water pump remained operable during this event, therefore the health and safety of the public would not have been affected.

3. Cause

It is suspected that material in the service water prevented the 10A pump check valve disc from properly seating. The unseated condition allowed backflow through the valve from the 10B pump reducing its discharge pressure.

4. Immediate Corrective Action

The 1-SW-P-10A service water pump was jogged and the check valve was observed to properly reseal. The service water pumps were returned to their normal condition with proper discharge pressure indicated on 1-SW-P-10B.

5. Subsequent Corrective Action

Pump 1-SW-P-10A was used to flush valve 1-SW-113. The valve was cycled to ensure its operability and was placed in service.

6. Action Taken to Prevent Recurrence

None.

7. Generic Implications

On January 9, 1982, a similar event occurred on Unit 2. In this event, the valve would not reseal and it was replaced.

Vepco

USNRC REGION II
ATLANTA, GEORGIA

83 AUG 18 4:49

AUG 15 1983

VIRGINIA ELECTRIC AND POWER COMPANY

Surry Power Station
P. O. Box 315
Surry, Virginia 23883

Serial No: 83-058

Docket No: 50-280

License No: DPR-32

Mr. James P. O'Reilly
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly

Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

Report Number

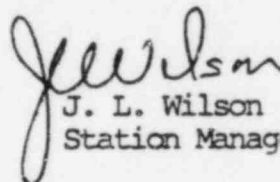
83-032/03L-0

Applicable Technical Specification

T. S. 6.6.2.b(2)

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,


J. L. Wilson
Station Manager

Enclosure

cc: Document Control Desk, USNRC
016 Phillips Bldg.
Washington, D. C. 20555

