

CONTROL BLOCK:

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 (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	C	A	S	0	S	2	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5				
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36				
Z		LICENSEE CODE														LICENSE NUMBER										LICENSE TYPE					CAT		58

CON T

REPORT SOURCE L 6 0 5 0 0 0 3 6 1 7 1 2 0 9 8 2 8 0 8 0 2 8 3 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 While in Mode 1, it was observed that two snubbers which are components of pipe
03 supports S2-FW-189-H013 were damaged and declared inoperable in accordance with
04 Limiting Condition for Operation (LCO) 3.7.6. Subsequently, on 12/22/82 while
05 the Unit was in Mode 5, three additional snubbers were observed to be damaged
06 and declared inoperable. Public health and safety were not affected by this
07 event.

08 _____

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE					
0	9	H	H	E	B	S	U	P	O	R	T	D	Z				
7	8	9	10	11	12	13	14	15	16	17	18	19	20				
LER/PO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.							
17	8	2	1	6	5	27	0	3	X	2							
21	22	23	24	25	26	27	28	29	30	31							
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
C	X	Z	Z	0	0	0	0	0	Y	N	A	P	0	2	9		
32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47		

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 It is now believed that the snubbers may have been initially damaged during

1 1 startup testing, which included a water hammer transient in March, 1981. The

1 2 engineering analysis required by Technical Specification 4.7.6.g was performed

1 3 with satisfactory results. All damaged snubbers have been replaced.

1 4

FACILITY STATUS: 1 5 B (28)
 % POWER: 0 5 0 (29)
 OTHER STATUS: N/A (30)
 METHOD OF DISCOVERY: C (31)
 DISCOVERY DESCRIPTION: Operator Observation (32)

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 [Z] (33) [Z] (34) N/A

N/A

PERSONNEL EXPOSURES		DESCRIPTION	
NUMBER	TYPE		
17	000	37	38
			N/A

PERSONNEL INJURIES
NUMBER: DESCRIPTION: (41)

1	8	0	0	0	(40)	N/A	8308220264 830802
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1		9		2		42		N/A		PDR ADOCK 05000361 S PDR		IE22	
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8		9		10				
ISSUED						PUBLICATION DESCRIPTION		(45)
2	0	N	(44)	N/A				
							NRC USE ONLY	

NAME OF PREPARER

N/A
H. B. RAY

PHONE (714) 492-7700

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 125

SAN CLEMENTE, CALIFORNIA 92672

H. B. RAY
STATION MANAGER

RECEIVED
NRC
1983 AUG -5 PM 12:39

SCE

REGION V LAR

TELEPHONE
(714) 592-7700

August 2, 1983

U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Marila Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J.B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361
Licensee Event Report No. 82-165, Revision 2
San Onofre Nuclear Generating Station, Unit 2

Reference: Letter, H. B. Ray (SCE) to R. H. Engelken (NRC),
dated January 7, 1983,
"Licensee Event Report No. 82-165"

The referenced Licensee Event Report (LER) reported the inoperability of two snubbers which are components of a pipe support (S2-FW-189-H013) located downstream of the main feedwater check valve to Unit 2 Steam Generator E088, inside the containment at the 63-1/2 foot level. These two snubbers were declared inoperable in connection with the dispositioning of Nonconformance Report (NCR) 4-032.

Limiting Condition for Operation (LCO) 3.7.6 requires these snubbers to be operable in Modes 1 thru 4. The associated Action Statement requires that the snubbers be replaced or restored to operable status within 72 hours and an engineering evaluation be performed in accordance with Technical Specification Section 4.7.6.g.

As part of the required engineering evaluation, additional snubbers were examined, found damaged and two additional NCR's (NCR 2-068 and NCR 2-069) were issued and three additional snubbers declared inoperable: one of these snubbers was on pipe support 2FW-189-H010 and the other two snubbers are on pipe support 2FW-189-H017. As a result of administrative oversight, the referenced LER was not revised until now to reflect these additional inoperable snubbers.

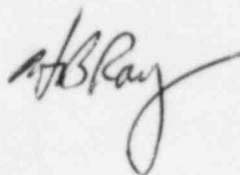
44
IE-22
83-317

August 2, 1983

The referenced LER indicated that the exact cause of the damaged snubbers was not known, however, it is now believed that they may have been damaged during startup testing, which included a water hammer transient in March 1981. A total of seven snubbers were damaged. Damage to two of the snubbers was apparent by visual inspection and they were replaced in January 1982 prior to fuel loading of Unit 2. Damage to the other five snubbers was not apparent until December 1982. These five snubbers are addressed in this LER, as revised. Analysis has demonstrated that the feedwater piping was not overstressed. All damaged snubbers have been replaced. Revision 2 to LER 82-165, reflecting the additional damaged snubbers and conclusions with respect to the cause of damage is enclosed.

Public health and safety were not affected by this event. If there are any questions regarding the above, please contact me.

Sincerely,



Enclosure: LER 82-165

cc: A.E. Chaffee (USNRC Resident Inspector, Units 2 and 3)
J.P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

U.S. Nuclear Regulatory Commission
Division of Technical Information and Document Control

Institute of Nuclear Power Operations (INPO)