

LICENSEE EVENT REPORT

CONTROL BLOCK:

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 (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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CONT

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During normal operation on 07/10/83, a step decrease of approximately
03 9% occurred in the output of steam flow transmitter 2FT-474. This feeds
04 one of two channels which provide a reactor trip should a steam flow-
05 feed flow mismatch occur coincident with low steam generator level.
06 This event is reportable in accordance with Technical Specification
07 15.6.9.2.B.1.

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REPORT
NUMBEREVENT YEAR
8 3SEQUENTIAL
REPORT NO.
0 0 9OCCURRENCE
CODE
0 3REPORT
TYPE
LREVISION
NO.
0ACTION
TAKEN
A 18 Z 19EFFECT
ON PLANT
Z 20SHUTDOWN
METHOD
Z 21HOURS
0 0 0 0ATTACHMENT
SUBMITTED
Y 23NPRD-4
FORM SUB.
Y 24PRIME COMP.
SUPPLIER
N 25COMPONENT
MANUFACTURER
F 1 8 0 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 This event was caused by a sudden nonconservative shift in the output
11 of an internal amplifier. The defective amplifier was replaced with a
12 spare and the transmitter was recalibrated and returned to service.
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14

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FACILITY
STATUS

% POWER

OTHER STATUS

METHOD OF
DISCOVERY

DISCOVERY DESCRIPTION

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RELEASED OF RELEASE

AMOUNT OF ACTIVITY

LOCATION OF RELEASE

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PERSONNEL EXPOSURES

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PERSONNEL INJURIES

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LOSS OF OR DAMAGE TO FACILITY

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PUBLICITY

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NAME OF PREPARER

C. W. Fay

PHONE

414/277-2811

ATTACHMENT TO LICENSEE EVENT REPORT NO. 83-009/03L-0

Wisconsin Electric Power Company
Point Beach Nuclear Plant Unit 2
Docket No. 50-301

On July 10, 1983, at 0430 hours, during normal operation, a step decrease of 0.2 to 0.3×10^6 lb/hr in the "B" loop main steam flow rate was discovered on the strip chart recorder. The bistables for flow transmitter 2FT-474 were immediately placed in the tripped position per ICP 10.2. This event is estimated to have gone undetected for approximately 10 minutes. The transmitter was still operable, but the excessive instrument error would have required flow rates slightly outside of Technical Specification limits to initiate protective actions. This transmitter and a similar redundant transmitter are used coincident with other parameters to initiate steam line isolation per Technical Specification Table 15.3.5.4, and also to initiate a reactor trip per Technical Specification Table 15.3.5.2. The minimum operability requirements for steam line isolation were not violated, since there is no minimum redundancy requirement. However, there is a minimum redundancy requirement of one for steam flow-feed flow mismatch coincident with low steam generator level. Although operable, the redundant channel would have required a mismatch slightly greater than the Technical Specification limit of 1×10^6 lb/hr in order to trip the bistable. This constitutes a reportable event in accordance with Technical Specification 15.6.9.2.B.1 as a "reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the Technical Specifications, but which do not prevent the fulfillment of the functional requirements of affected systems."

The cause of the nonconservative instrument error was found to be a faulty amplifier internal to the transmitter. This amplifier was one of many just recently installed during the refueling outage in order to meet environmental qualification requirements. The amplifier had been in service for approximately one month prior to failure. The amplifier was replaced with an identical spare, Foxboro part No. BO142EC, and the channel was returned to service at 1230 hours on July 10, 1983. The faulty amplifier was returned to the factory for determination of cause of failure and repair.

No other failures with the newly installed amplifiers have been found.

The Resident Inspector has been notified of this event.



Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

August 3, 1983

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NO. 50-301
LICENSEE EVENT REPORT NO. 83-009/03L-0
POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report No. 83-009/03L-0
(a 30-day report) which provides a description of an event
reportable in accordance with Technical Specification 15.6.9.2.B.1,
"Reactor protection system or engineering safety feature instrument
settings which are found to be less conservative than those
established by the Technical Specifications, but which do not
prevent the fulfillment of the functional requirements of affected
system."

Very truly yours,

Executive Vice President

Sol Burstein

Enclosure

Copy to NRC Resident Inspector

AUG 8 1983

1E22 1/1