

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

V A S P S 1 2 0 0 - 0 0 0 0 0 0 - 0 0 0 3 4 1 1 1 1 4 5
8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CONT
0 1 REPORT SOURCE L 6 0 5 0 0 0 0 2 8 0 7 0 7 1 0 8 3 8 0 8 1 0 8 3 9
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 On July 10, 1983, with Unit No. 1 at 100% power, containment temperature exceeded
0 3 120°F for approximately 3 hours. This is contrary to T.S.-3.8, Figure 3.8/1 and
0 4 is reportable per T.S.6.6.2.b.(2). Other containment parameters remained within
0 5 specifications and the maximum temperature deviation was less than 1%. Therefore,
0 6 the health and safety of the public were not affected.
0 7
0 8

0 9 SYSTEM CODE Z Z 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE Z Z Z Z Z 14
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
17 LER/RO REPORT NUMBER 8 3 0 3 1 0 3 L 0
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
Z 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 N 24 N 25 Z Z Z Z 26
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 The cause was due to incorrect data being supplied by Unit No. 2 computer which
1 1 was used to compute Unit No. 1 containment temperature. Subsequent to the event,
1 2 correct data was used to monitor containment temperature.
1 3
1 4

1 5 FACILITY STATUS E 28 % POWER 1 0 0 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Operator Observation 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
1 6 ACTIVITY RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE N/A 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION N/A 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION N/A 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
2 0 PUBLICITY ISSUED N 44 DESCRIPTION N/A 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

8308190364 830810
PDR ADOCK 05000280
S PDR

1022
11

NRC USE ONLY

ATTACHMENT 1
SURREY POWER STATION, UNIT NO. 1
DOCKET NO: 50-280
REPORT NO: 83-031/03L-0
EVENT DATE: 07-10-83

TITLE OF THE EVENT: CONTAINMENT TEMPERATURE HIGH

1. DESCRIPTION OF THE EVENT:

With Unit 1 at 100% power, it was discovered that Unit 1 containment temperature exceeded 120°F for approximately 3 hours. This is contrary to T.S. 3.8 Figure 3.8.1 and is reportable per Technical Specification 6.6.2.b.(2).

2. PROBABLE CONSEQUENCES AND STATUS OF REDUNDANT EQUIPMENT:

Calculations for the design of the containment spray system, which depressurizes the containment in the unlikely event of a LOCA, are based on a maximum initial dry bulb containment temperature of 120°F. During the time containment temperature exceeded 120°F, containment pressure and RWST temperature were at levels which were more conservative than those used in the Technical Specification design basis. Since the maximum containment temperature was less than 1% above the T.S. limit for a short period of time, the health and safety of the public were not affected.

3. CAUSE:

Following the failure of Unit No. 1 process computer, the Unit No. 2 computer was used to collect the necessary data to compute the Unit No. 1 containment temperature. Some of the data provided by the Unit No. 2 computer was incorrect which led to a computed containment temperature less than the actual containment temperature. The inaccurate computer temperature points were not properly accounted for in the weighted average temperature computations.

The 'B' containment air recirculation fan had been stopped to set proper blade pitch and thus provide improved containment cooling during high seasonal temperature periods. While the recirc. fan was secured, containment temperature unknowingly exceeded 120°F. The incorrect data being supplied by Unit No. 2 computer led the operators to believe that containment temperature was being maintained less than 120°F.

4. IMMEDIATE CORRECTIVE ACTION:

At the time it was discovered that Unit No. 1 containment temperature had previously exceeded 120°F, the containment temperature was less than 120°F, therefore, no immediate corrective action was necessary.

5. SUBSEQUENT CORRECTIVE ACTION:

After verifying that the Unit No. 1 computer was accurate, Unit No. 1 computer was used to monitor Unit No. 1 containment temperature.

ATTACHMENT 1

SURRY POEWR STATION, UNIT NO. 1

DOCKET NO: 50-280

REPORT NO: 83-013/03L-0

EVENT DATE: 07-10-83

TITLE OF THE EVENT: CONTAINMENT TEMPERATURE HIGH

6. ACTION TAKEN TO PREVENT RECURRENCE:

None.

7. GENERIC IMPLICATIONS:

None.

Vepco

USNRC REGION II
ATLANTA, GEORGIA

83 AUG 17 A 8:06

AUG 10 1983

VIRGINIA ELECTRIC AND POWER COMPANY

Surry Power Station
P. O. Box 315
Surry, Virginia 23883

Serial No: 83-057

Docket No: 50-280

License No: DPR-32

Mr. James P. O'Reilly
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly

Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

Report Number

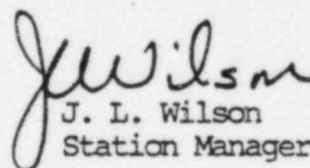
83-031/03L-0

Applicable Technical Specification

T. S. 6.6.2.b(2)

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

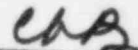

J. L. Wilson
Station Manager

Enclosure

cc: Document Control Desk, USNRC
016 Phillips Bldg.
Washington, D. C. 20555

DESIGNATED ORIGINAL

Certified By



IE 22

11