

3150-0011

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	A	L	J	M	F	1	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5			
7	8	9					14		15										25		26								30	37	CAT	39
LICENSE CODE								LICENSE NUMBER												LICENSE TYPE												

01 REPORT SOURCE L 6 0 5 0 0 0 3 4 8 7 0 5 2 5 8 3 8 0 8 1 0 8 3 9

0 2 | At 0610 on 5/25/83, following an increased count rate on containment atmosphere
0 3 | particulate monitoring system R-11, RCS unidentified leakage was determined to be
0 4 | 1.75 gpm per FNP-1-STP-9.0 (RCS Leakage Test). Tech. Spec. 3.4.7.2, in part, re-
0 5 | quires unidentified leakage to be limited to 1 gpm. Tech. Spec. 3.4.7.2 action
0 6 | statement requirements were met. Health/safety of the public was not affected.
0 7 |
0 8 |

SYSTEM CODE		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE				COMP. SUBCODE	VALVE SUBCODE				
0	9	C	B	X	Z	V	A	L	V	E	X	F	P
7	8	9	10	11	12	13	14	15	16	17	18	19	20
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.			
17	8	3	—	0	2	4	/	0	3	X	—	1	
21	22	23	24	25	26	27	28	29	30	31	32		
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER					
E	Z	B	Z	0	Y	N	N	K	0	8	5		
33	34	35	36	37	40	41	42	43	44	45	46	47	48

At 1513 on 5/25/83 the leakage was identified as being past the RCS "B" loop drain valve into the reactor coolant drain tank. The LCO was cleared and the "B" loop drain valve was tightened. At 1925 on 5/25/83, the unidentified leakage was determined to be .12 gpm. (See Attachment).

FACILITY STATUS		% POWER		OTHER STATUS (30)		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION (32)		
1	5	E	(28)	1	0	0	(29)	NA	A (31)	Operational Event

ACTIVITY CONTENT RELEASED OF RELEASE: 1 6 2 33 34 AMOUNT OF ACTIVITY NA LOCATION OF RELEASE NA

PERSONNEL EXPOSURES									
NUMBER				TYPE		DESCRIPTION			
1	7	0	0	0	(37)	Z	(38)	NA	(39)

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	8	0	0	0	NA

LOSS OF OR DAMAGE TO FACILITY
TYPE DESCRIPTION (43) NA

1 9 Z (42) 10

8308190331 830810
PDR AD0CK 05000348
S PDR

PUBLICITY
ISSUED DESCRIPTION (45)

2 0 N (44) NA

7 8 9 10 68

NRC USE ONLY

69 70 71 72 73 74 75 76 77 78 79 80

PHONE: (205) 899-5156

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONTINUED)

Note: This attachment provides additional information concerning this incident.

Upon discovery of an apparent RCS leak into containment, a check revealed an unidentified leak rate of 1.75 GPM. Containment sump indication revealed that the leakage was into the sump. The On-Call Operations Manager and Emergency Director were notified by the shift supervisor and preparations were made to enter the containment. An inspection of containment (outside the biological shield) revealed no leakage. Several systems (portions of letdown and charging) were isolated in an unsuccessful attempt to isolate the leakage source. As a result, it was determined that the leakage was occurring inside the biological shield.

Additional planning was initiated for an entry inside the biological shield. A review of containment radiation survey data vs. reactor power levels was performed and it was determined that a containment entry should be made at 35% reactor power in order to ensure personnel safety and meet ALARA concerns.

Following a power reduction to 35%, an entry was made inside the biological shield. The "A" RCDT pump suction valve was observed to be leaking at the body to bonnet joint. The valve was closed thereby stopping the leak. Since the "A" RCDT pump suction line was hot to the touch, the loop drain valves from the RCS loops to the RCDT pump suction line were also checked. The "B" RCS loop drain valve was determined to be slightly off of its seat and at that point the leakage was reclassified as "identified" and the LCO cleared. The valve was tightened and at 1925 on 5/25/83, the RCS unidentified leakage was determined to be .12 GPM.