LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1 | ILLS C 1 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 0 0 0 (4) | | (5)
8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

VT
1 1 | REPORT SOURCE L (6) | 0 5 0 0 0 3 7 3 (7) | 0 7 1 1 8 3 (8) | 0 8 1 1 8 3 (9)
8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

1 2 | On July 18, 1983 during a reactor scram with reactor water low level at -50", a
1 3 | Primary Containment Isolation System (PCIS) Group II isolation occurred. Valve
1 4 | 1CM027 which is a PCIS Group II isolation valve indicated open and failed to close with
1 5 | the isolation signal present. The failure of the Group II isolation valve 1CM027 to
1 6 | close was discovered by the operator and immediately closed. Operation of the plant
1 7 | was not affected.
1 8 |
8 9 80

1 9 | SYSTEM CODE S D (11) | CAUSE CODE E (12) | CAUSE SUBCODE A (13) | COMPONENT CODE V A L V E X (14) | COMP. SUBCODE X (15) | VALVE SUBCODE D (16)
8 9 10 11 12 13 18 19 20

(17) LER/RO REPORT NUMBER: EVENT YEAR 8 3 | — (23) | SEQUENTIAL REPORT NO. 0 8 4 (24 26) | / (27) | OCCURRENCE CODE 0 3 (28 29) | REPORT TYPE L (30) | — (31) | REVISION NO. 0 (32)

ACTION TAKEN Z (18) | FUTURE ACTION Z (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS (22) 0 0 0 0 (37 40) | ATTACHMENT SUBMITTED Y (23) | NPRD-4 FORM SUB. N (24) | PRIME COMP. SUPPLIER A (25) | COMPONENT MANUFACTURER V 0 3 1 0 (26)
33 34 35 36 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The failure of valve 1CM027 to close could not be determined. The valve when tested
1 1 | functioned flawlessly when PCIS signals were simulated and the control switch cycled.
1 2 | Work Request (L26060) was written to investigate and correct the problem. The valve
1 3 | tested satisfactorily. The valve 1CM027 was tested again with the unit at rated condi-
1 4 | tions under work request (L26109). No problem could be found. The valve is made by
8 9 | Valcor. 80

FACILITY STATUS B (28) | % POWER 0 0 0 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY A (31) | DISCOVERY DESCRIPTION (32) Observation
1 5 | 9 10 12 13 44 45 46 80

ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA
1 6 | 9 10 11 44 45 80

PERSONNEL EXPOSURES: NUMBER 0 0 0 (37) | TYPE Z (38) | DESCRIPTION (39) NA
1 7 | 9 11 12 13 80

PERSONNEL INJURIES: NUMBER 0 0 0 (40) | DESCRIPTION (41) NA
1 8 | 9 11 12 80

LOSS OF OR DAMAGE TO FACILITY: TYPE Z (42) | DESCRIPTION NA
1 9 | 9 10 80

IE22
1/1

PUBLICITY: ISSUED N (44) | DESCRIPTION (45)
2 0 | 9 10

8308190275 830811
PDR ADOCK 05000373
S PDR

NRC USE ONLY
68 69 80

NAME OF PREPARER Vincent Masterson PHONE: 815/357-6761

GPU 81-228

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

August 11, 1983

James G. Keppler
Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Clen Ellyn, IL 60137

Dear Sir:

Reportable Occurrence Report #83-084/03L-0 Docket #050-373 is being submitted to your office in accordance with LaSalle County Nuclear Power Station Technical Specification 6.6.B.2.(b), conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

C E Sargent
for G. J. Diederich
Superintendent
LaSalle County Station

GJD/GW/rg

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U. S. NRC Document Management Branch
INPO-Records Center
File/NRC

IE22

AUG 15 1983

I. LER NUMBER: 83-084/03L-0

II. LASALLE COUNTY STATION: Unit 1

III. DOCKET NUMBER: 050-373

IV. EVENT DESCRIPTION:

On July 18, 1983 during a reactor scram with reactor low water level at -50", a Primary Containment Isolation System (PCIS) Group II isolation occurred. Valve 1CM027 which is a PCIS Group II isolation valve indicated open and failed to close with an isolation signal present. The valve is required to close on a Group II isolation. Technical Specification 3.6.3 was exceeded. Upon cycling of the valve electrically with its control switch, the valve closed.

V. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

At the time of the occurrence the plant was in the startup mode with Reactor power at 4%. Electro Hydraulic Control (EHC) troubleshooting was in progress. It was this troubleshooting on the EHC system which resulted in a level transient and subsequent scram. Level in the reactor vessel decreased to -50" initiating the High Pressure Core Spray and Reactor Core Isolation Cooling (HPCS and RCIC) along with Primary Containment Isolation System (PCIS) Groups I through V.

The failure of the Group II PCIS isolation valve 1CM027 to close isolating the drywell continuous air sampling lines was discovered by the operator and immediately closed. Operation of the plant was not affected.

VI. CAUSE:

The failure of valve 1CM027 to close could not be duplicated. The valve when tested functioned flawlessly when PCIS signals were simulated and the control switch cycled. A reason for the failure of valve 1CM027 to close could not be determined.

VII. CORRECTIVE ACTION:

Work Request (L26060) was written to investigate and correct the problem.

The valve was tested and closed satisfactorily in response to simulated PCIS signals and the cycling of its control switch. No apparent cause could be determined.

The valve 1CM027 was tested again with the unit at rated conditions (greater than 25% power) under Work Request (L26109). The valve and its position indication functioned properly in response to simulated PCIS signals. Work Request (L26109) was completed on August 4, 1983. The valve is made by VALCOR.

Prepared by: Vincent Masterson