

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
1 1 L Q A D 1 2 0 0 0 - 0 0 0 - 0 0 0 3 4 1 1 1 1 4 1 5
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

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7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
0 1 REPORT SOURCE L 6 0 5 0 0 0 2 5 4 7 0 1 7 1 5 8 1 3 8 0 8 0 4 8 1 3 9
DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
0 2 At 1550 hours, on July 15, 1983, the RHR return to Suppression Chamber valve, M0
0 3 1001-36A, was taken out of service for repairs. This rendered the 'A' loop of
0 4 Containment Cooling inoperable. All required surveillances were performed as stated
0 5 in Technical Specification 4.5.B.3. The redundant 100% capacity 'B' loop of
0 6 Containment Cooling, both Core Spray Loops, and the Unit One Diesel Generator were
0 7 operable at all times; therefore, the safety implications of this event were
0 8 minimal.
7 8 97 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE
C F 11 E 12 A 13 C K T B R K 14 A 15 Z 16
17 LER RC REPORT NUMBER 8 3 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED NPRD-4 FORM SUB PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
C 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 Y 24 N 25 G 0 8 0 26
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
1 0 Testing of the valve motor operator circuit breaker revealed that the instantaneous
1 1 trip setting on one phase was too low, causing the breaker to trip intermittently
1 2 during valve operation. The setting could not be adjusted; and the breaker was
1 3 replaced with a newer model circuit breaker. The valve was tested satisfactorily
1 4 and the 'A' Containment Cooling loop was declared
operable at 2225 hours the same day.
7 8 97 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
1 5 FACILITY STATUS 28 E 29 0 9 7 29 30 NA 31 B 32 Monthly Surveillance Test
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
1 6 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 NA LOCATION OF RELEASE 36 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 807 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
1 7 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 807 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
1 8 PERSONNEL INJURIES NUMBER DESCRIPTION 41 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 807 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
1 9 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 43 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 807 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
2 0 PUBLICITY ISSUED DESCRIPTION 45 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

NAME OF PREPARER C Iben

PHONE 309-654-2241, ext 173

8308190224 830804
PDR ADOCK 05000254
S PDR

NRC USE ONLY

- I. LER NUMBER: LER/RO 83-28/03L-0
- II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station
- III. FACILITY NAME: Unit One
- IV. DOCKET NUMBER: 050-254
- V. EVENT DESCRIPTION:

On July 15, 1983, at 3:50 p.m., the RHR return to Suppression Chamber valve, MO 1-1001-36A, was taken out of service, in the closed position, to investigate its intermittent breaker trips which occurred during the performance of QOS 1000-3 (LPCI Motor Operator Valve Operability Test). This action rendered the 1A loop of RHR Containment Cooling System inoperable. Thus, Technical Specification 3.5.B.1.a was not satisfied. All required surveillances for an inoperable Containment Cooling loop were performed prior to taking the valve out of service.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

In anticipation of the MO 1-1001-36A valve being taken out of service, the operability tests of the 1B RHR Containment Cooling Loop, both A and B Core Spray pumps, and the Unit One Diesel Generator were performed successfully. This satisfies Technical Specification 4.5.B.3. surveillance requirements in the event that one loop of RHR Containment Cooling becomes inoperable. Thus, no adverse safety implications existed during the event.

VII. CAUSE:

Subsequent testing by the Electrical Maintenance Department revealed that the instantaneous trip setting on one phase of the motor feed breaker was too low (about 85 amperes). This caused the breaker (General Electric Model TEF136M1025) to intermittently trip during valve operation.

VIII. CORRECTIVE ACTION:

The instantaneous trip point for the one phase of the TEF breaker could not be raised to the proper instantaneous trip setting of 100 amperes. Therefore, the TEF breaker was replaced with a G.E. Model TEC 36030 breaker with the instantaneous trip settings adjusted to 105 amperes. The MO 1-1001-36A valve was subsequently stroked three times to verify operability and was returned to service at 10:25 p.m. the same day.



Commonwealth Edison

Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

NJK-83-274

August 4, 1983

J. Keppier, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One
Appendix A, Section 3.5.B.3

Enclosed please find Reportable Occurrence Report Number RO 83-28/03L-0
for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of
Technical Specification 6.6.B.2.b; operation in a degraded mode permitted
in a limiting condition for operation.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center

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