

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT
/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/9/ (7) /0/7/0/4/8/3/ (8) /0/8/0/3/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On July 4, 1983, during a periodic test, the 2H Emergency Diesel generator /
/0/3/ / tripped on high jacket coolant temperature. The 2H EDG was declared inoperable /
/0/4/ / and the redundant EDG was verified operable as per T.S. 4.8.1.1.2.a.4 and two /
/0/5/ / independent offsite AC power sources were verified operable as per T.S. /
/0/6/ / 4.8.1.1.1.a within one hour. Because the redundant EDG and the offsite AC power /
/0/7/ / supplies were available the health and safety of the general public were not /
/0/8/ / affected. This event is reportable pursuant to T.S. 6.9.1.9.b. /

SYSTEM CAUSE CAUSE COMP. VALVE
CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE

/0/9/ /E/E/ (11) /X/ (12) /Z/ (13) /E/N/G/I/N/E/ (14) /Z/ (15) /Z/ (16)
LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION
(17) REPORT NO. NO.
NUMBER /8/3/ /-/ /0/5/4/ /-/ /0/3/ /L/ /-/ /0/

ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER

/X/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /Y/ (23) /N/ (24) /A/ (25) /F/0/1/0/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / After running loaded for 118 minutes the EDG tripped on high jacket coolant /
/1/1/ / temperature apparently caused by low lube oil level. Lube oil was added and the /
/1/2/ / diesel was allowed to cool prior to retesting. The PT was repeated approximately /
/1/3/ / six hours after the trip and the diesel ran for greater than two hours without /
/1/4/ / any further problems. /

FACILITY METHOD OF
STATUS %POWER OTHER STATUS DISCOVERY DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28) /1/0/0/ (29) / NA / (30) /B/ (31) / Surveillance Test /

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES
NUMBER TYPE DESCRIPTION (39)
/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES
NUMBER DESCRIPTION (41)
/1/8/ /0/0/0/ (40) / NA /

LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
/1/9/ /Z/ (42) / NA /

PUBLICITY
ISSUED DESCRIPTION (45)
/2/0/ /N/ (44) / NA /

NRC USE ONLY

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

Virginia Electric and Power Company
North Anna Power Station, Unit No. 2
Docket No. 50-339
Attachment to LER 83-054/03L-0

Attachment: Page 1 of 2

Description of Event

On July 4, 1983, during a periodic test, the "2H" Emergency Diesel Generator (EDG) tripped on high jacket coolant temperature. The diesel had been started by a simulated loss of offsite power signal using 2-PT-82.2A ("2H" Diesel Generator Test). The diesel was loaded to greater than 2750 kWe at 1109. The trip occurred at 1307, which is two minutes less than the two hour run time required by the PT. Technical Specification 4.8.1.1.2.a.5 requires only a one hour run, however to prevent prelube oil fires on the exhaust manifold, two hour runs are required by this procedure. The "2H" EDG was declared inoperable and the "2J" EDG started within one hour to verify operability. Also, two independent circuits between the offsite transmission network and the onsite Class 1E Distribution System were verified operable within one hour. This event is contrary to T.S. 3.8.1.1 and reportable pursuant to T.S. 6.9.1.9.b.

Probable Consequences of Occurrence

The consequences of this event are limited because the redundant Emergency Diesel Generator was verified operable and two offsite AC sources were available. Also, high jacket coolant temperature would not have tripped "2H" EDG if an emergency start signal was present. As a result, the health and safety of the general public were not affected.

Cause of Event

The diesel tripped on high jacket coolant temperature apparently caused by low lube oil level. Immediately after the trip the diesel jacket coolant temperature from the engine was 208F (trip occurs at 205F) and coolant to the engine was 184F.

The simulated loss of offsite power signal was present only during the diesel start. No automatic emergency start signals were present when the diesel tripped. If an auto emergency start signal had been present, then, high jacket coolant temperature would not have caused an emergency shutdown of the diesel.

Immediate Corrective Action

Lube oil was added and the diesel was allowed to cool prior to retesting. The PT was repeated approximately six hours after the diesel tripped (diesel started at 1855) and the diesel ran for greater than two hours without any further problems. Jacket coolant temperature was monitored and remained below 185°F.

Scheduled Corrective Action

No scheduled corrective action is required.

Action Taken To Prevent Recurrence

No further action is required to prevent recurrence.

Generic Implications

No generic implications resulted from this event.

Vepco

USNRC REGION II
ATLANTA, GEORGIA

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

83 AUG 11 A10:38

August 3, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-104
NO/RST: 11
Docket No. 50-339
License No. NPF-7

Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No.

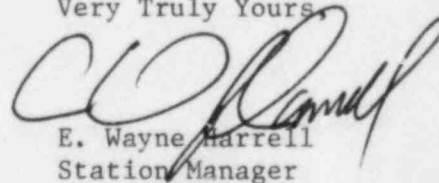
Applicable Technical Specifications

LER 83-054/03L-0

T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours


E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
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U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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