

# OPERATING DATA REPORT

DOCKET NO. 50-266

DATE May 9, 1983

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

## OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 1
  2. REPORTING PERIOD: APRIL 1983
  3. LICENSED THERMAL POWER (MWT): 1518.
  4. NAMEPLATE RATING (GROSS MWE): 523.8
  5. DESIGN ELECTRICAL RATING (NET MWE): 497.
  6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
  7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
  8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:  
NOT APPLICABLE
  9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): 390.0
  10. REASONS FOR RESTRICTIONS, (IF ANY): Power level restricted because of self-imposed hot leg limitation in an attempt to limit steam generator tube corrosion.
- |   | THIS MONTH | YR TO DATE | CUMULATIVE  |
|---|------------|------------|-------------|
| 11. HOURS IN REPORTING PERIOD               | 719        | 2,879      | 109,415     |
| 12. NUMBER OF HOURS REACTOR WAS CRITICAL    | 719.0      | 2,830.6    | 90,402.9    |
| 13. REACTOR RESERVE SHUTDOWN HOURS          | 0.0        | 1.4        | 625.4       |
| 14. HOURS GENERATOR ON LINE                 | 719.0      | 2,823.9    | 87,932.2    |
| 15. UNIT RESERVE SHUTDOWN HOURS             | 0.0        | 0.0        | 793.5       |
| 16. GROSS THERMAL ENERGY GENERATED (MWH)    | 840,721    | 3,288,526  | 119,237,515 |
| 17. GROSS ELECTRICAL ENERGY GENERATED (MWH) | 281,740    | 1,096,480  | 39,972,270  |
| 18. NET ELECTRICAL ENERGY GENERATED (MWH)   | 267,116    | 1,039,072  | 38,020,790  |
| 19. UNIT SERVICE FACTOR                     | 100.0      | 98.1       | 80.4        |
| 20. UNIT AVAILABILITY FACTOR                | 100.0      | 98.1       | 81.1        |
| 21. UNIT CAPACITY FACTOR (USING MDC NET)    | 75.1       | 72.9       | 71.1        |
| 22. UNIT CAPACITY FACTOR (USING DER NET)    | 74.8       | 72.6       | 69.9        |
| 23. UNIT FORCED OUTAGE RATE                 | 0.0        | 0.2        | 2.7         |
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
Twenty-six week refueling and steam generator replacement outage scheduled to commence September 30, 1983.
  25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN

DOCKET NO. 50-266

UNIT NAME Point Beach Unit 1

DATE May 9, 1983

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH April, 1983

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>372</u>	11	<u>371</u>	21	<u>371</u>
2	<u>374</u>	12	<u>373</u>	22	<u>371</u>
3	<u>371</u>	13	<u>371</u>	23	<u>372</u>
4	<u>373</u>	14	<u>372</u>	24	<u>373</u>
5	<u>372</u>	15	<u>372</u>	25	<u>372</u>
6	<u>373</u>	16	<u>371</u>	26	<u>372</u>
7	<u>373</u>	17	<u>372</u>	27	<u>371</u>
8	<u>373</u>	18	<u>371</u>	28	<u>372</u>
9	<u>372</u>	19	<u>372</u>	29	<u>371</u>
10	<u>359</u>	20	<u>371</u>	30	<u>372</u>
				31	<u>          </u>

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April, 1983

DOCKET NO. 50-266  
UNIT NAME Point Beach Unit 1  
DATE May 9, 1983  
COMPLETED BY C. W. Fay  
TELEPHONE 414/277-2811

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting <sup>3</sup> Down Reactor	Licensee Event Report No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective Action To Prevent Recurrence

1 F: Forced  
S: Scheduled

2 Reason:

A- Equipment Failure (explain)  
B- Maintenance or Test  
C- Refueling  
D- Regulatory Restriction  
E- Operator Training & License Exam  
F- Administrative  
G- Operational Error (explain)  
H- Other (explain)

3 Method:

1- Manual  
2- Manual Scram  
3- Automatic Scram  
4- Other (explain)

4 Exhibit G-Instruc-

tions for Preparation of Data Entry  
Sheets for LER File  
(NUREG-0161)

5 Exhibit I- Same

Source

## NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-266  
Unit Name Point Beach Unit 1  
Date May 9, 1983  
Completed By C. W. Fay  
Phone 414/277-2811

Unit 1 operated at approximately 371 MWe net throughout the period with no major load reductions. The primary-to-secondary leakage remains stable at less than ten gallons per day.

On April 5, 1983 at 0820 hours the fire detection system for the north half of the cable spreading room was found inoperable. At 1219 hours the system was restored and put back in operation.

At approximately 0615 hours on April 6, 1983 contractor personnel inadvertently tripped three circuit breakers inside of control panel C01 while setting up to pull wires to complete NRC-mandated fire protection backfits. Unit 1 received a common critical control power failure alarm as control room personnel heard molded circuit breakers trip. Breakers involved were Nos. 151, 152, and 153 which power valve 1CV-3200B, 1B04-86 lockout relay, and 4D emergency diesel fuel tank level control. The breakers were immediately reset and the control board returned to normal.

This event was regarded as reportable under Technical Specification 15.6.9.2.A.6.

# OPERATING DATA REPORT

DOCKET NO. 50-301

DATE May 9, 1983

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

## OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 2
2. REPORTING PERIOD: APRIL 1983
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:  
NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	719	2,879	94,200
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	2,003.6	84,061.7
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	196.8
14. HOURS GENERATOR ON LINE	0.0	1,992.9	82,648.1
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	181.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	0	3,007,255	114,522,849
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0	1,009,260	38,810,660
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0	963,699	36,952,666
19. UNIT SERVICE FACTOR	0.0	69.2	87.7
20. UNIT AVAILABILITY FACTOR	0.0	69.2	87.9
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	67.6	79.8
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	67.4	78.9
23. UNIT FORCED OUTAGE RATE	0.0	0.0	1.5
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): NONE			

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: JUNE 30, 1983

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

DOCKET NO. 50-301

UNIT NAME Point Beach Unit 2

DATE May 9, 1983

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH April, 1983

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>-2</u>	11	<u>-2</u>	21	<u>-2</u>
2	<u>-2</u>	12	<u>-2</u>	22	<u>-2</u>
3	<u>-2</u>	13	<u>-2</u>	23	<u>-2</u>
4	<u>-2</u>	14	<u>1*</u>	24	<u>-2</u>
5	<u>-2</u>	15	<u>-2</u>	25	<u>-2</u>
6	<u>-2</u>	16	<u>-2</u>	26	<u>-2</u>
7	<u>-2</u>	17	<u>-2</u>	27	<u>-2</u>
8	<u>-2</u>	18	<u>-2</u>	28	<u>-2</u>
9	<u>-2</u>	19	<u>-2</u>	29	<u>-2</u>
10	<u>-2</u>	20	<u>-2</u>	30	<u>-2</u>
				31	<u></u>

\*Erroneous results due to unexplained  
meter change caused by relay testing.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-301

UNIT NAME Point Beach Unit 2

DATE May 9, 1983

REPORT MONTH April, 1983

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective Action To Prevent Recurrence
1	830325	S	719	C	1	N/A	ZZ	ZZZZZZ	Completed Week 6 of the continuing 14-week refueling and steam generator sleeving outage.

<sup>1</sup> F: Forced  
S: Scheduled

<sup>2</sup> Reason:  
A- Equipment Failure (explain)  
B- Maintenance or Test  
C- Refueling  
D- Regulatory Restriction  
E- Operator Training & License Exam  
F- Administrative  
G- Operational Error (explain)  
H- Other (explain)

<sup>3</sup> Method:  
1- Manual  
2- Manual Scram  
3- Automatic Scram  
4- Other (explain)

<sup>4</sup> Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

<sup>5</sup> Exhibit I- Same Source



# NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-301  
Unit Name Point Beach Unit 2  
Date May 9, 1983  
Completed By C. W. Fay  
Phone 414/277-2811

Unit 2 was shut down during the entire report period as it completed the sixth week of the scheduled 14-week refueling and steam generator sleeving outage.

Outage-related work included the removal of "A", "B", and "C" moisture separator reheater tube bundles for changeout, the removal of 4A and 4B feedwater heaters for changeout, the rebuilding of "A" reactor coolant pump, the commencement of safeguards relay changeouts, reconditioning of the "C" component cooling water heat exchanger, and the changeout and overhaul of both low-pressure turbine rotors. Research and repairs are under way regarding the steam erosion experienced in turbine casings and steam piping. Inspection and minor repairs were initiated on residual heat removal valves 2MOV-700 and 2MOV-701 and to reactor temperature detection manifold isolation valves 2RC-559A and 2RC-559B.

While performing refueling leakage tests of containment isolation valves on April 9, 1983, the "A" reactor coolant pump component cooling water supply containment isolation valve (755A) was found to have leakage such that the limit in Technical Specification 15.4.4.III.B may have been exceeded. During the initial phase of the type "C" test, pressurization of the test volume to the required test pressure could not be achieved. Upon failing to obtain test pressure, it was noted that the subject valve was leaking as evidenced by the flow of air through a test connection used to provide a leakage flow path. At this time, prior to obtaining a leakage rate reading, the testing personnel tapped on the subject valve in an attempt to seat it. The attempt was successful as no signs of air flow through the leakage path was then evident. This action by testing personnel prevented the meaningful quantification of the as-found leakage. This event is reportable in accordance with Technical Specification 15.6.9.2.A.3, "Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment."

Three relays which provide undervoltage protection to safeguards switchgear 2A05 and 2A06 were discovered out of Technical Specification tolerances during testing. A 30-day Licensee Event Report will be filed with the NRC.



On April 28, 1983 it was evaluated that valve 559A, "A" loop RTD bypass manifold valve isolation valve, body-to-bonnet studs were found degraded due to corrosion and constituted an abnormal degradation of the RCS boundary.

During performance of refueling outage activities, a higher than normal contamination of an area in close proximity to the valve was noted. An investigation by Maintenance personnel as to the possible cause of the contamination found the valve with signs of body-to-bonnet leakage. Four of twelve studs were found corroded and are undergoing further evaluation. All studs will be replaced and the similar valve in the other loop has been inspected with no degradation found. This has been reported in accordance with Technical Specification 15.6.9.A.3.

During the period, Health Physics took added precautions as an iodine airborne problem became evident after the reactor vessel head was removed and the core was unloaded.

During the period, eddy current testing and a secondary-to-primary leak check was completed in the steam generators. The leak check revealed three explosive plugs which appeared wet and three tubes which were dripping or wet in the "A" steam generator and one explosive plug and three tubes which appeared wet in the "B" steam generator. Eddy current testing data revealed 16 tubes with indications of >40% repair limit in the "A" steam generator and two tubes with indications >40% in the "B" steam generator. Degraded tubes will be repaired either by sleeving or mechanical plugging and degraded plugs will be weld repaired. The NRC was notified of the tube degradation with Licensee Event Report No. 83-002/01T-0.

Additionally, the sleeving program scheduled during this outage, designed to prevent further steam generator tube degradation, is well under way with approximately 54% of the work completed in the "B" steam generator.