



**Florida  
Power**  
CORPORATION

May 9, 1983  
3F-0583-07

Director of Nuclear Reactor Regulation  
Attention: Mr. John F. Stolz, Chief  
Operating Reactors Branch No. 4  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: Crystal River Unit 3  
Docket No. 50-302  
Operating License No. DPR-72  
NUREG-0737, Item II.K.3.30  
Revised Small-Break Loss-of-Coolant-Accident Methods  
to Show Compliance with 10 CFR 50, Appendix K

Dear Sir:

On November 1, 1982, Babcock & Wilcox Company submitted topical reports BAW-10092P, Revision 3 (CRAFT2-FORTRAM Program for Digital Simulation of a Multinode Reactor Plant) and BAW-10154P (B&W Small Break LOCA ECCS Evaluation Model). Florida Power Corporation endorses these reports as our response to the requirements of the subject NUREG Item. It is our understanding that work on NUREG-0737, Item II.K.3.30 should not be started until NRC approval is received for these two reports.

Sincerely,

G. R. Westafer  
Manager  
Nuclear Licensing and Fuel Management

RMB:mm

cc: Mr. James P. O'Reilly  
Regional Administrator, Region II  
Office of Inspection & Enforcement  
U.S. Nuclear Regulatory Commission  
101 Marietta Street N.W., Suite 2900  
Atlanta, GA 30303

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