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Writer's Direct Dial Number:

May 9, 1983
5211-83-149

Office of Nuclear Reactor Regulation
Attn: John F. Stolz, Chief
Operating Reactors Branch No. 4
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit 1 (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Technical Specification Change Request No. 125 and
Request for Approval of Steam Generator Repair

In accordance with my letter dated April 19, 1983, we are submitting our request for a change in the TMI-1 Technical Specification 4.19, which would permit the operation of TMI-1 following repair of the steam generators by any methods other than plugging, provided that the repair methods are shown to be acceptable by analysis and approved by the NRC. Enclosed are three signed originals and forty copies of Technical Specification Change Request No. 125 requesting amendment to Appendix A of Operating License No. DPR-50. Also enclosed is one signed copy of Certificate of Service for proposed Technical Specification Change Request No. 125 to the chief executives of the township and county in which the facility is located.

We have previously submitted to the NRC detailed descriptions and analyses of our steam generator repair method and information supporting its adequacy. Accordingly, we also request approval of this repair method under the provisions of the proposed Technical Specification revision, to permit both a non-nuclear heatup of the plant using pump heat for precritical testing, and subsequent operation using the repaired steam generators. We propose that the necessary approval as well as the change in Technical Specifications be treated as license amendments for purposes of the recently adopted regulations to implement the Snolly Amendment.

NRC approval of the OTSG repair method involves review of a complex issue. Accordingly, we will forward a check for \$12,300 under separate cover as required pursuant to 10CFR 170.22 for a Class IV required approval.

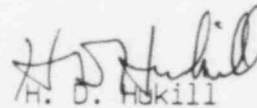
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Pursuant to 10CFR 50.91(a)(1), we enclose our analyses, using the standards in 10CFR 50.92, of significant hazards considerations both of the proposed Technical Specification change and of operation with the repaired steam generators. Pursuant to 10CFR 50.91(b) of the regulations, we have provided a copy of this letter, the proposed change in Technical Specifications, and our analyses of significant hazards considerations to Thomas Gerusky, the designated representative of the Commonwealth of Pennsylvania.

We request that upon receipt of this letter, NRC immediately publish in the Federal Register a notice of receipt of both our request for a Technical Specification change and of our request for approval of the heat-up and subsequent operation of TMI-1 using repaired steam generators. We are aware that such a notice would be published together with a notice of opportunity for interested persons to request a public hearing and with the NRC Staff's preliminary determination of significant hazards considerations. We are also aware that in connection with any approval to operate TMI-1 with repaired steam generators, NRC may establish license conditions based on the steam generator repair, surveillance and test programs. To preclude any unwarranted delays in issuing these license conditions and thus permitting the return to service of the steam generators, we would like to further request that these license conditions also be included in the Federal Register notice at least in summary form.

Sincerely,



H. D. Hskill
Vice President TMI-1

HDH/klk
enclosure

cc: W. Dornsife
J. Van Vliet