



BOSTON EDISON

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

E. S. Kraft, Jr.

Vice President Nuclear Operations
and Station Director

February 11, 1994
BECO Ltr. #94-018

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

License No. DPR-35
Docket No. 50-293

JANUARY 1994 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

E. S. Kraft, Jr.
E. S. Kraft Jr.

WJM/dmc/9407

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE February 11, 1994
 COMPLETED BY: W. Munro
 TELEPHONE (508) 830-8474

OPERATING STATUS

NOTES

1. Unit Name Pilgrim I
2. Reporting Period January 1994
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>185376.0</u>
12. Number of Hours Reactor Was Critical	<u>744.0</u>	<u>744.0</u>	<u>113685.5</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>744.0</u>	<u>109533.0</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1478496.0</u>	<u>1478496.0</u>	<u>192623232.0</u>
17. Gross Electrical Energy Generated (MWH)	<u>511480.0</u>	<u>511480.0</u>	<u>65157414.0</u>
18. Net Electrical Energy Generated (MWH)	<u>492681.0</u>	<u>492681.0</u>	<u>62626375.0</u>
19. Unit Service Factor	<u>100.0</u>	<u>100.0</u>	<u>59.1</u>
20. Unit Availability Factor	<u>100.0</u>	<u>100.0</u>	<u>59.1</u>
21. Unit Capacity Factor (Using MDC Net)	<u>98.8</u>	<u>98.8</u>	<u>50.4</u>
22. Unit Capacity Factor (Using DER Net)	<u>101.1</u>	<u>101.1</u>	<u>51.6</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>11.7</u>
24. Shutdowns scheduled over next 6 months (type, date, and duration of each) - <u>None</u>			
25. If shutdown at end of report period, estimated date of startup - <u>Unit operating</u>			

AVERAGE DAILY UNIT POWER LEVEL

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COMPLETED BY: W. Munro
TELEPHONE (508) 830-8474

MONTH January 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	665	17	666
2	667	18	666
3	665	19	667
4	666	20	667
5	665	21	667
6	664	22	667
7	663	23	668
8	601	24	667
9	620	25	667
10	665	26	666
11	663	27	665
12	665	28	665
13	666	29	667
14	665	30	666
15	666	31	666
16	665		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR JANUARY 1994

The unit started the reporting period at 100 percent core thermal power (CTP) and this power level was essentially maintained for the entire reporting period. There was a brief power reduction on January 8, 1994 to approximately 47 percent to facilitate a main condenser backwash.

SAFETY RELIEF VALVE CHALLENGES
MONTH OF JANUARY 1994

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 date 01/05/81.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: April 1, 1995
3. Scheduled date for restart following next refueling: May 26, 1995
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1993 refueling outage was of the same design as loaded in the previous refueling outage and consisted of 140 assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1629 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
 - (b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 691 fuel assemblies.

MONTH JANUARY 1994

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
DIESEL GENERATORS AND AUXILIARIES	"B" EMERGENCY DIESEL GENERATOR X-107B. (PR94.9033)	FAILED SURVEILLANCE ACCEPTANCE CRITERIA CONCERNING STARTING TIME. PROCEDURE 8.9.1	TURBO AIR ASSIST SOLENOID VALVES SV4570B AND SV4569B FAILED IN CLOSED POSITION DUE TO MOISTURE RESIDUE IN THE VALVES.	DISASSEMBLED SOLENOID VALVES, CLEANED AND REASSEMBLED. PERFORMED PROCEDURE 8.9.1 SATISFACTORILY.	TO BE DETERMINED.	N/A
SALT SERVICE WATER (SSW) SYSTEM	"A" SSW PUMP P-208A DISCHARGE CHECK VALVE 29-CK-3880A (PR94.9023)	CHECK VALVE FAILED IN THE CLOSED POSITION	ROOT CAUSE UNDER INVESTIGATION	DISASSEMBLED VALVE. INSPECTED VALVE AND INTERNALS. REPLACED LEAKING HINGE PIN COPPER GASKETS (4) WITH GRAPHITE GASKETS. (DCN 94-89723). POST WORK TESTING COMPLETED SATISFACTORILY.	WHEN REMAINING SSW CHECK VALVES ARE DISASSEMBLED, COPPER GASKETS WILL BE REPLACED WITH GRAPHITE GASKETS.	N/A
STANDBY LIQUID CONTROL (SBLC) SYSTEM	"A" AND "B" SBLC PUMPS P-207A AND B	PUMPS LEAKING	WORN PACKING	REMOVED AND REPLACED EXISTING PACKING ON BOTH PUMPS WITH CHEVRON TYPE PACKING PER FRN 93-03-77. DRAINED AND REPLACED OIL, AND CHANGED OUT CRANKCASE COVER GASKET. THE THREE PLUNGERS IN PUMP P-207A WERE REPLACED (THEY WERE SLIGHTLY PITTED AND CORRODED).	NONE.	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS DOCKET NO: 50-293

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TELEPHONE: (508) 830-8474
REPORT MONTH JANUARY, 1994

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
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There were no Unit Shutdowns or reportable Power Reductions during the reporting period.

1	2	2	3	4&5
F-FORCED S-SCHED	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)