

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/9/ (7) /0/4/0/2/8/3/ (8) /0/4/2/1/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On April 2, 1983, with Unit 2 in Mode 3, a periodic test of the Main Steam Safety/
/0/3/ / Valves revealed that the pressure setpoints of valves SV-MS-201C, 202C, 203A and /
/0/4/ / and C, 204A and 205A were less than the required setpoints of T.S. 3.7.1.1 by /
/0/5/ / greater than 1 percent. Since the remaining safety valves were operable and the /
/0/6/ / affected valves would have lifted to prevent secondary overpressurization, the /
/0/7/ / health and safety of the public were not affected. This event is reportable /
/0/8/ / pursuant to T.S. 6.9.1.9.b. /

SYSTEM CAUSE CAUSE COMP. VALVE
CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE

/0/9/ /C/C/ (11) /E/ (12) /B/ (13) /V/A/L/V/E/X/ /J/ /B/
LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION
REPORT NO. CODE TYPE NO.
(17) NUMBER /8/3/ /-/ /0/2/7/ / / /0/3/ /L/ /-/ /0/

ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER

/E/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /Y/ (23) /N/ (24) /A/ (25) /C/7/1/0/
(26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The cause of this event was setpoint drift. The pressure setpoints of the af- /
/1/1/ / fected safety valves were adjusted and the valves were tested with satisfactory /
/1/2/ / results. /
/1/3/ / /
/1/4/ / /

FACILITY METHOD OF
STATUS %POWER OTHER STATUS DISCOVERY DISCOVERY DESCRIPTION (32)
/1/5/ /G/ (28) /0/0/0/ (29) / NA / (30) /B/ (31) / Surveillance Test /

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES
NUMBER TYPE DESCRIPTION (39)
/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES
NUMBER DESCRIPTION (41)
/1/8/ /0/0/0/ (40) / NA /

LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
/1/9/ /Z/ (42) / NA /
PUBLICITY

ISSUED DESCRIPTION (45)

/2/0/ /N/ (44) / NA /

NAME OF PREPARER E. WAYNE HARRELL

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NRC USE ONLY

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PDR ADOCK 05000339
S PDR

Virginia Electric and Power Company
North Anna Power Station, Unit No. 2
Docket No. 50-339
Report No. LER 83-027/03L-0

Attachment: Page 1 of 1

Description of Event

On April 2, 1983, while performing a periodic test on the Unit 2 Main Steam Safety Valves, it was discovered that the pressure setpoints of the following safety valves were lower than required by T.S. 3.7.1.1.

<u>Mark No.</u>	<u>As Found Setpoint</u>	<u>Required Setpoint</u>
SV-MS-201C	1072 psig	1085 psig \pm 1%
SV-MS-202C	1071 psig	1095 psig \pm 1%
SV-MS-203A	1091 psig	1110 psig \pm 1%
SV-MS-203C	1085 psig	1110 psig \pm 1%
SV-MS-204A	1082 psig	1120 psig \pm 1%
SV-MS-205A	1119 psig	1135 psig \pm 1%

This event is reportable pursuant to T.S. 6.9.1.9.b.

Probable Consequences of Occurrence

The operation of the Main Steam Safety Valves is required to protect the steam generators and main steam piping against overpressurization. Since the failed safety valves would have opened if required to provide this protection and the remaining safety valves were operable, the health and safety of the public were not affected.

Cause of Event

The cause of this event was setpoint drift. The safety valves are spring loaded nozzle type, style HA-65, size 6R-10 manufactured by the Crosby Valve and Gage Company.

Immediate Corrective Action

The pressure setpoints of the failed valves were adjusted and the valves were subsequently tested with satisfactory results.

Scheduled Corrective Action

No further action is required.

Actions Taken to Prevent Recurrence

No further action is required.

Generic Implications

There are no generic implications associated with this event.