LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | TNSNP1 (7) | 00-00000-00 (3) | 4111 (4) | (5)
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T

01 | REPORT SOURCE L (6) | 05000327 (7) | 032883 (8) | 042683 (9)
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | Unit 1 in mode 3 at 0% Rx power. During the review of surveillance instruction (SI)
03 | 111, it was discovered that on 09/11/82, 5 out of 10 main steam safety relief valves
04 | (1-RV-1-512, -513, -515, -518, and -521) did not meet required setpoint criteria and
05 | during the last half of SI-111 performance on 01/08/83, 2 out of 10 main steam safety
06 | relief valves (1-RV-1-525 and -527) did not meet required setpoint criteria. This
07 | condition could have existed in applicable modes causing entry into LCO 3.7.1.1.
08 | There was no effect on public health or safety. Previous occurrences - one (SQRO-50-
7 8 9 327/83001). 8

09 | SYSTEM CODE HB (11) | CAUSE CODE E (12) | CAUSE SUBCODE B (13) | COMPONENT CODE VALVEX (14) | COMP. SUBCODE P (15) | VALVE SUBCODE B (16)
7 8 9 10 11 12 13 18 19 20

(17) LER/RO REPORT NUMBER: EVENT YEAR 83 | — | SEQUENTIAL REPORT NO. 048 | / | OCCURRENCE CODE 03 | REPORT TYPE L | — | REVISION NO. 0
21 22 23 24 26 27 28 29 30 31 32

ACTION TAKEN E (18) | FUTURE ACTION G (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS 0000 (22) | ATTACHMENT SUBMITTED Y (23) | NPRD-4 FORM SUB. N (24) | PRIME COMP. SUPPLIER N (25) | COMPONENT MANUFACTURER C710
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | The 6" x 10" Crosby valves (Model No. 6xRx10-HA-65-FN) were from .18% to 1.1% out of
11 | acceptable ranges. Slight drifting of the setpoints was caused by normal aging of the
12 | valve spring and ambient temperature variations. The failed valves were adjusted to
13 | within acceptable limits immediately after discovery. Valve performance will be
14 | surveyed during future tests to determine if periodic spring replacement would reduce
7 8 9 failures. Additionally, SI-111 will be revised to clarify reporting requirements. 80

FACILITY STATUS D (28) | % POWER 000 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY A (31) | DISCOVERY DESCRIPTION (32) Surveillance Review
15 7 8 9 10 12 13 44 45 46 80

ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA
16 7 8 9 10 11 44 45 80

PERSONNEL EXPOSURES
NUMBER 000 (37) | TYPE Z (38) | DESCRIPTION (39) NA
17 7 8 9 11 12 13 80

PERSONNEL INJURIES
NUMBER 000 (40) | DESCRIPTION (41) NA
18 7 8 9 11 12 80

LOSS OF OR DAMAGE TO FACILITY (43)
TYPE Z (42) | DESCRIPTION NA
19 7 8 9 10 80

8305030579 830426
PDR ADOCK 05000327
S PDR

PUBLICITY
ISSUED N (44) | DESCRIPTION (45) NA
20 7 8 9 10 68 69 80

NRC USE ONLY

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Sequoyah Nuclear Plant

LER SUPPLEMENTAL INFORMATION

SQRO-50-327/83048 Technical Specification Involved: 3.7.1.1

Reported Under Technical Specification: 6.9.1.13.b

Date of Occurrence: 03/28/83 Time of Occurrence: 1300 CST

Identification and Description of Occurrence:

During th review of surveillance instruction SI-111 on 03/28/83, it was discovered that on 09/11/82, 5 out of 10 main steam safety relief valves (1-RV-1-512, -513, -515, -518, and -521) did not meet required setpoint criteria and during the last half of SI-111 performance on 01/08/83, 2 out of 10 main steam safety relief valves (1-RV-1-525 and -527) did not meet required setpoint criteria.

Conditions Prior to Occurrence:

Unit 1 in mode 3 at 0% power.

Apparent Cause of Occurrence:

The apparent causes were the normal aging of the valve spring and ambient temperature variations.

Analysis of Occurrence:

The 6" x 10" Crosby main steam safety valves (Model No. 6xRx10HA-65-FN) varied from .18% to 1.1% out of acceptable pressure ranges. Slight drifting of the setpoints of this type valve is normal due to ambient temperature variations and valve spring aging. SI-111 did not contain appropriate action to be taken for reportability requirements if a valve failed its setpoint criteria. This resulted in reporting this event after SI review was completed rather than at the time of occurrence.

Corrective Action:

Valve performance will be surveyed during future SI-111 performances to see whether periodic spring replacement would reduce the number and severity of setpoint failures. SI-111 will be revised to clarify action to be taken concerning reportability in the event of main steam safety relief valve performance failure. All failed valves were adjusted within acceptable limits and satisfactorily tested immediately after discovery.

Failure Data:

None.