

New Hampshire Yankee

Ted C. Feigenbaum
President and
Chief Executive Officer

NYN- 91090

May 30, 1991

United States Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Document Control Desk

References: (a) Facility Operating License No. NPF-86, Docket No. 50-443
(b) PSNH Letter (SBN-1211) dated October 9, 1986 "10CFR50.59
Evaluation", G.S. Thomas to V.S. Noonan

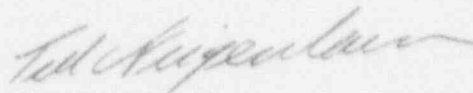
Subject: 10CFR50.59 Quarterly Report

Gentlemen:

Enclosed please find the Quarterly Report of 10CFR50.59 Safety Evaluations for Seabrook Station. This report covers the period of January 1, 1991, to March 31, 1991, and is being submitted pursuant to the reporting requirements outlined in Reference b).

Should you require further information regarding this matter, please contact Mr. James M. Peschel, Regulatory Compliance Manager at (603) 474-9521, extension 3772.

Very truly yours,



Ted C. Feigenbaum

Enclosure(s)

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PDR ADOCK 05000443
R PDR

New Hampshire Yankee Division of Public Service Company of New Hampshire
P.O. Box 300 • Seabrook, NH 03874 • Telephone (603) 474-9521

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United States Nuclear Regulatory Commission
Attention: Document Control Desk

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cc: Mr. Thomas T. Martin
Regional Administrator
United States Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Mr. Gordon Edison, Sr. Project Mgr.
Project Directorate I-3
Division of Reactor Projects
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Noel Dudley
NRC Senior Resident Inspector
P.O. Box 1149
Seabrook, NH 03874

New Hampshire Yankee
May 30, 1991

ENCLOSURE 1 TO NYN-91090

Seabrook Station
10CFR50.59 Safety Evaluation
Quarterly Report
January 1, 1991 - March 31, 1991

1. Design Changes

The below listed design changes were implemented at Seabrook Station and safety evaluations were performed pursuant to the requirements of 10CFR50.59.

Design Coordination Report: Number 86-332

Title: Sound Powered Telephone Jack for the Fuel Transfer Control Panel

Description: This Design Coordination Report (DCR) adds a sound powered telephone jack to the Fuel Transfer Control Panel to enable direct communication between the Control Room and the refueling station. This communication link is required per Technical Specification 3/4.9.5.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Design Coordination Report: Number 86-357

Title: New and Spent Fuel Rack Tolerance Corrections and Access Ladder for Cask Handling Crane

Description: This Design Coordination Report (DCR) installs self leveling devices in certain new fuel rack storage cells to bring the base levelness to Westinghouse specifications. This DCR also implements administrative controls to ensure that spent fuel assemblies which are stored in spent fuel rack cells which deviate from Westinghouse specifications are not inserted in the Reactor Vessel. Installation of the self leveling devices does not affect the performance or function of the fuel rack.

This DCR also installs a seismically anchored ladder to improve access to the cask handling crane.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Design Coordination Report: Number 86-728

Title: Snubber Installation

Description: This Design Coordination Report (DCR) adds snubbers to the process sensing lines of various pressure instruments in the Radioactive Gaseous Waste System, Fuel Storage Building HVAC, Reactor Makeup Water System, and the Containment Enclosure Cooling and Exhaust Filter System. The instruments located in these systems were experiencing

pressure fluctuations or spiking, due to either compressor or fan operations and in some cases wind currents on the outside atmospheric reference leg to the instruments. These snubbers will eliminate fluctuations/spiking causing inaccurate process representation and potential instrument damage.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Design Coordination Report: Number 87-062

Title: Interferences Due to Thermal Expansion

Description: This Design Coordination Report (DCR) requires notching of the Turbine Building Floor Steel and Conduit Pullbox Support to allow the counterweight on valve EX-V-026 unrestricted movement. Interference between these components occurs due to thermal expansion. This modification does not compromise the structural integrity of the support.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Design Coordination Report: Number 87-224

Title: New Fuel Elevator Modifications

Description: This Design Coordination Report (DCR) provides modifications to the New Fuel Elevator to improve its reliability. These modifications include the replacement of turnbuckle pins with nuts and bolts locked with cotter pins to provide a more secure locking arrangement.

Final Safety Analysis Report Figures 9.1-4 and 9.1-5 are affected by this DCR.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question. Changes to the Updated Final Safety Analysis Report will be incorporated by means of a future revision.

Design Coordination Report: Number 89-071

Title: Additional Steam Trap in Water Treatment Area

Description: This Design Coordination Report (DCR) provides for the installation of a steam trap station in the Auxiliary Steam System in the water

treatment room. This steam trap will remove condensate collected at the end of the steam header which heats the caustic acid heat exchanger, to eliminate the potential for water hammer in branch connections. The condensate from this trap is returned to the Auxiliary Steam Condensate System.

The portions of the Auxiliary Steam and Auxiliary Steam Condensate Systems affected by this change are non-safety related.

Conclusion:

A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Design Coordination Report: Number 90-012

Title: Electric Manholes Inspection Ports

Description:

This Design Coordination Report (DCR) installs removable 4 1/2 inch Bisco Fire Plugs in certain electric manhole plugs to facilitate inspection and dewatering. This installation does not compromise the structural integrity of the manhole plugs.

Conclusion:

A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

2. Minor Modifications

The below listed minor modifications were made at Seabrook Station and safety evaluations were performed pursuant to the requirements of 10CFR50.59.

Minor Modification: Number 91-0504

Title: Indication for Loss of Control Power SI-V93

Description:

This Minor Modification (MMOD) revises Final Safety Analysis Report Figure 8.3-45 to correct the wiring schematic diagram for Safety Injection valve SI-V93 to reflect the actual installation. The installed wiring configuration provides the intended function and the circuit is correct as presently wired.

Conclusion:

A safety evaluation was performed for this change, and it was determined that this change will not create an unreviewed safety question. Changes to the Updated Final Safety Analysis Report will be incorporated by means of a future revision.

Minor Modification: Number 90-0649

Title: Oil/Water Separator Vault Temperature Switch Replacement

Description: This Minor Modification (MMOD) replaces the temperature switches with equivalent switches from a different manufacturer. These switches are non-safety related but are necessary for freeze protection. Problems have been experienced with the existing switches and exact replacements are no longer available.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Minor Modification: Number 90-0620

Title: Turbine Building Hydrogen Cylinder Storage Rack

Description: This Minor Modification (MMOD) installs a storage rack for five additional hydrogen cylinders in the Turbine Building. This allows for the storage of a total of eleven hydrogen cylinders for turbine generator cooling. This modification meets the hydrogen storage requirements specified in the National Fire Protection Association Code. Additionally, the Fire Hazard Analysis for the Turbine Building contained in the Seabrook Station fire Protection Program, Evaluation and Comparison to BTP APCSB 9.5-1, Appendix A, is unaffected by this change. The worst case fire generated by the stored hydrogen is less severe than the design basis fire for this area of the Turbine Building.

The Wet Sprinkler System in the subject portion of the Turbine Building will also be modified by this MMOD to relocate two existing sprinkler heads and to provide one additional head.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Minor Modification: Number 89-574

Title: Reorientation of Ball Drip Valve

Description: This Minor Modification (MMOD) reorients Fire Protection System Ball drip valve 1-FP-V8000 by installing a 90 degree elbow and nipple. This modification will allow the valve and associated fire protection system piping to drain, thus ensuring adequate freeze protection.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Minor Modification Number: 90-0661

Title: Revision of Pipe Support 52-SG-2

Description: This Minor Modification (MMOD) removes the bottom portion of pipe support 52-SG-2 which supports a 3/4" seal water supply pipe. The removed portion of the pipe support impeded free movement of the Reactor Coolant Pump C pump seal replacement davit arm. The modified support will still comply with the original design requirements.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Minor Modification: Number 90-656

Title: Blowdown Flash Tank Inlet Piping Material Change

Description: This Minor Modification (MMOD) replaces the carbon steel piping upstream of the steam generator blowdown flash tank inlet nozzles with stainless steel piping of the same configuration. The stainless steel piping will provide better resistance to erosion than the carbon steel piping. The affected portion of the blowdown system is non-safety related.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

Minor Modification: Number 90-0512

Title: Handrails for RHR Vault Platform Access Openings

Description: This Minor Modification (MMOD) installs handrails to completely enclose ladder access openings located in the ~ 16 foot elevation platform of Residual Heat Removal (RHR) Vaults 1 and 2. This MMOD will enhance personnel safety.

Conclusion: A safety evaluation was performed for this design change, and it was determined that this change will not create an unreviewed safety question.

3. Temporary Modifications

The below listed temporary modifications were made at Seabrook Station and safety evaluations were performed pursuant to the requirements of 10CFR50.59.

Temporary Modification: Number 91-003

Description: This Temporary Modification (TMOD) removed from service, three carbon monoxide sensors which were installed to provide early warning of a fire within the containment charcoal filter beds (CAH-F8). The CO sensors have been declared inoperable and fire detection is currently being provided by Leak Detection System RTD's. The removal of the CO sensors from service eliminates a continuous control room trouble alarm.

Conclusion: A safety evaluation was performed for this TMOD and it was determined that an unreviewed safety question was not created.

Temporary Modification: Number 91-004

Description: This Temporary Modification (TMOD) provides a flowpath from the Steam Generator Blowdown Demineralizers to the CST to help maintain secondary plant water inventory after a plant trip or during plant start-up while the start up feed pump is in service taking suction from the CST.

This modification also provides a flush connection and sample point allowing the Steam Generator Blowdown Demineralizers effluent piping to be flushed to the turbine building sump after a regeneration. This will ensure removal of residual chemicals that may be left in the piping after a regeneration so they can not be carried forward to the steam generators.

Conclusion: A safety evaluation was performed for this TMOD and it was determined that an unreviewed safety question was not created.

Temporary Modification: Number 91-002

Description: This Temporary Modification (TMOD) provides controlled drawings which reflect the partial implementation and operation of a new bulk hydrogen storage facility.

Generator cooling requirements will be supplied by the new storage facility while primary side hydrogen requirements will be supplied via the existing pressure control station and piping until the design change is fully implemented.

Conclusion: A safety evaluation was performed for this TMOD and it was determined that an unreviewed safety question was not created.

4. Technical Requirements Manual

The below listed Technical Requirements Manual changes have been approved for use and safety evaluations were performed pursuant to the requirements of 10CFR50.59.

Technical Requirements Manual Change Request: Number 91-01

Title: Snubber Visual Inspection Intervals

Description: This Technical Requirement Manual Change Request modifies Technical Requirement 5 to reflect alternative requirements for snubber visual inspection intervals in accordance with Generic Letter 90-09.

Conclusion: A safety evaluation was performed for this Technical Requirement Manual change and it was determined that it does not introduce an unreviewed safety question.

Technical Requirements Change Request: Number 90-09

Title: Fire Pump Diesel Engine Surveillance

Description: This Technical Requirement Change Request modifies Technical Requirement 7 by deleting the requirement that the 18 month inspection be performed during shutdown.

Conclusion: A safety evaluation was performed for this Technical Requirement Manual change and it was determined that it does not introduce an unreviewed safety question.

Technical Requirements Manual Change Request: Number 89-05

Title: Deletion of Inaccessible Fire Detectors

Description: This Technical Requirement Manual Change Request modifies Technical Requirement 12 by deleting eleven inaccessible ionization detectors in the Fuel Storage Building. Four infra-red flame detectors were provided to provide fire detection for this area. The infra-red detectors can be tested from the floor without the installation of scaffolding.

Conclusion: A safety evaluation was performed for this Technical Requirement Manual change and it was determined that it does not introduce an unreviewed safety question.

5. Final Safety Analysis Report

The below listed Final Safety Analysis Report (FSAR) change requests were issued and safety evaluations were performed pursuant to the requirements of 10CFR50.59

- FCR 91-023, Updated FSAR Chapter 1
- FCR 91-026, Updated FSAR Chapter 3
- FCR 90-117, Updated FSAR Chapter 6
- FCR 91-010, Updated FSAR Chapter 7
- FCR 91-022, Updated FSAR Chapter 8
- FCR 90-001, Updated FSAR Chapter 9
- FCR 90-109, Updated FSAR Chapter 10
- FCR 91-015, Updated FSAR Chapter 11
- FCR 90-121, Updated FSAR Chapter 12
- FCR 90-102, Updated FSAR Chapter 15
- FCR 90-104, Updated FSAR Chapter 16

New Hampshire Yankee has committed to submit an Updated FSAR pursuant to 10CFR50.71(e) by May 26, 1991. The Updated FSAR chapters are currently being prepared. Each Updated FSAR chapter is required to be supported by a safety evaluation which addresses the Updated FSAR changes from the existing FSAR. These changes which will be summarized in the Updated FSAR transmittal result primarily from the following:

Design changes and modifications recently implemented and made operable.

Incorporation of responses to NRC Requests for Additional Information and other information previously submitted.

Elimination of Unit 2 references.

Editorial Changes.

Thus far, safety evaluations have been completed for the above listed Updated FSAR Chapters and it was determined that the changes involved do not constitute an unreviewed safety question.

6. Applications to Amend the Operating License

The below listed applications to amend the Seabrook Station Operating License submitted pursuant to 10CFR50.90 have been determined not to involve a Significant Hazard Consideration per 10CFR50.92 or an Unreviewed Safety Question per 10CFR50.59:

- Request for License Amendment; Increased Enrichment of Reload Fuel Assemblies,

(NYN-91049, dated 03/18/91).
- Request for License Amendment; Definition of Digital Channel Operational Test,

(NYN-91010, dated 01/24/91).
- Supplement 1 to Application to Amend Facility Operating License No. NPF-86 to Authorize North Atlantic Energy Service Company to act as Managing Agent for Seabrook Station, Unit No. 1,

(NYN-91005, dated 01/15/91).
- Request for License Amendment; Residual Heat Removal System Isolation Valve Autoclosure Interlock Removal,

(NYN-91011, dated 01/24/91).

7. Procedures

There were no procedures developed during this reporting period that required safety evaluations pursuant to 10CFR50.59.

8. Procedure Changes

There were no procedure changes made during this reporting period that required safety evaluations in accordance with 10CFR50.59.

9. Tests or Experiments

There were no tests or experiments performed during this reporting period that required safety evaluations in accordance with 10CFR50.59.

10. Justifications for Continued Operations (JCO)

There were no JCO's written during the reporting period.