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April 13, 1983
5211-83-094

Regional Administrator
Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Sir:

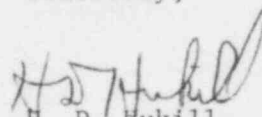
Three Mile Island Nuclear Station, Unit 1 (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
1982 Annual Report, Additional Information

Our letter 5211-83-058, dated February 28, 1983, forwarded to you the personnel exposure data and aircraft movement data for our 1982 Annual Report. This report should also have included the information from the Leak Reduction Program tests (per Section 6.9.1.B.3 of the TMI-1 Tech Specs), and information on the pressurizer power operated relief valve and pressurizer safety valve challenges (per Section 6.9.1.B.4 of the TMI-1 Tech Specs). This information was not included with submittal 5211-83-058 due to an administrative oversight, and we are, therefore, sending it at this time.

Attachment I is a description of the leak reduction program, and the tests done for the calander year 1982. *

There is no information to report on the pressurizer power operated relief valve and pressurizer safety valve challenges, as TMI-1 was in an extended cold shutdown condition during 1982.

Sincerely,


H. D. Hukill
Director, TMI-1

HDH:JRP:JGB:vjf

Attachment

cc: Director, Office of Inspection & Enforcement (40 copies)
Director, Office of Management Information

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Annual Report Regarding the Periodic Leak Reduction Program Tests (T.S. 6.9.1.B.3)

The periodic leak reduction program is composed of the following plant surveillance procedures (the systems affected are as indicated):

<u>Surveillance Procedure No.</u>	<u>Affected System</u>
1303-11.16	Decay Heat Removal System
1303-11.18	R. B. Local Leak Rate Testing
1303-11.27	Makeup and Purification System
1303-11.28	Liquid Waste Disposal System
1303-11.29	Waste Gas Disposal System
1303-11.30	Reactor Coolant Sampling - Liquid & Gas
1303-11.31	Hydrogen Recombiner System
1303-11.50	Reactor Building Spray System

At the time of issuance of Restart Report Section 2.1.1.8, the leakage reduction program was still in the conceptual stage. Table 2.1-4 of the Restart report listed these systems which were thought to be within the scope of the NUREG-0578 Leakage Reduction Program at that time. Subsequent development of the leakage reduction program determined that the Fluid Block System would not be within the scope of the program because this system does not contain radioactive fluids when in normal operation and would not contain highly radioactive fluids during a serious transient or accident. Additionally, the Reactor Building Local Leakage Penetration Pressurization System is not within the scope of the program because it is an air system which pressurizes the penetrations to prevent out-leakage and does not contain radioactive gases.

Table I summarizes the results of the leakage Reduction Program tests and inspections by procedure number that were performed between January 1 and December 31, 1982. Component identification of those components found to be leaking and the type repair (if required) are included in Table I. Because of TMI-1's extended cold shutdown condition by NRC Order and for OTSG tube repairs, a complete battery of leak reduction program tests was not required to be completed in 1982. Containment integrity was not required during 1982 and only the Decay Heat Removal, Liquid Waste and Waste Gas Disposal Systems were required to be operable during 1982. A total leakage of 5320 cc/hr (all from the Building Spray System) was found; this leakage was successfully repaired and retested to no detectable leakage.

TABLE I

Surveillance Procedure No.	Date of Performance	Component Identification	Leakage (cc/hr or CFM gaseous)		Type Repair Performed Prior To As Left Test
			AS FOUND	AS LEFT	
1303-11.16	9/10/82 (Loop A)	N/A	None	0	N/A
	7/24/82 (Loop B)	N/A	None	0	N/A
1303-11.18		(See Note 1)			
1303-11.27		(See Note 2)			
1303-11.28	12/31/82	N/A	None	0	N/A
1303-11.29		(See Note 3)			N/A
1303-11.30		(See Note 4)			
1303-11.31	1/18/82 5/20/82	(See Note 5)	0	0	
1303-11.50	10/21/82 (AS FOUND) and 12/08/82 (AS LEFT)	BS-V-3B	(See Note 6) 360 cc/hr.	0	Repacked valve per Job Ticket #C9441
		BS-V-60A	1000 cc/hr.	0	Repacked & cleaned valve per Job Ticket #C9428
		BS-P-1A	3600 cc/hr.	0	Cleaned and sealed threaded joints per Job Ticket #C9430
		BS-P1B	360 cc/hr.	0	Cleaned and sealed threaded joints per Job Ticket #C8721

Note 1: Not required by Tech. Specs. to be performed in 1982. Currently being performed. Results will be reported as part of the 1983 annual report. Since a total of 86 components are tested as part of the Technical Specifications-required LR Testing procedure, for conciseness, only those components determined to require repair as a result of as-found leakage will be reported. Also details are covered by the ILRT reporting requirement to NRC. (Tech. Spec. 4.4.1.1)

Note 2: Not performed in 1982 due to long-term cold S/D plant conditions. Makeup system was not placed in service in 1982.

Note 3: Not performed in 1982. A leakage test was last performed 10/8/80. Leakage was less than 3.2×10^{-4} CFM and acceptable. Due to the requirement to perform this test after piping and valve replacement near WDG-V4 (Refer to LER 82-02), this surveillance has been deferred to 1983, prior to Unit 1 restart. Due to

TABLE I (CONTINUED)

extended (>3 years) shutdown conditions and the resulting low plant radioactivity levels, extension of the test interval has not increased significantly the likelihood or magnitude of off-site releases from this system.

Note 4: Not performed in 1982. Last performed 7/6/81. Zero leakage found. Requires hot plant conditions to complete in its entirety. These conditions could not be achieved in 1982 due to OTSG tube repairs in progress.

Note 5: The system is new and all piping except the "tie-in" piping to the spare recombiner was leak tested per Startup Test Procedure 250/3.3 on 1/18/82. The normal service Recombiner and its associated piping were leak tested per Startup TP #243/1 on 5/20/82. No leakage was identified during either test.

Note 6: Due to the extended (>3 years) shutdown conditions and plant radioactivity levels, the 2 hour site boundary and 30 day low population zone offsite dose considerations were not limiting conditions for acceptable leakage criteria in 1982. The Reactor Building Spray System was not required to be operable or in service during 1982. Therefore, the "as-found" leakage recorded did not significantly increase the likelihood or magnitude of either on or off-site releases during 1982.