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May 14, 1991  
C311-91-2054

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)  
Operating License No. DPR-50  
Docket No. 50-289  
Monthly Operating Report April 1991

Enclosed are two copies of the April, 1991 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

T. G. Broughton  
Vice President & Director, TMI-1

WGH:

Attachments

cc: Administrator, Region I  
TMI Senior Resident Inspector

9105170185 910430  
PDR ADOCK 05000289  
R PDR

OPERATIONS SUMMARY  
APRIL 1991

The unit entered the month operating at ~93% power producing ~810 MWe gross electrical generation. The A Reactor Coolant Pump Seal Leakoff flows continued to increase. The necessary procedure changes have been made and approved to continue pump operation. Communication with the vendor on the matter is ongoing. The unit exceeded 395 days of continuous operation. This is the longest period of continuous operation for a nuclear plant in the state of Pennsylvania. The unit completed the month operating at ~92% power producing ~780 MWe gross electrical generation. Power production continues to be limited by high OTSG level on the "B" side.

MAJOR SAFETY RELATED MAINTENANCE

During April, Main Steam Valve MS-V-15B was removed from service to determine the cause for the extremely hard turning stem and make necessary repairs. Inspection identified damage to the valve bonnet bearing caused by inadequate lubrication. A new bearing and grease seal were installed and the valve returned to service. The maintenance effort and parts replacement were duplicated on Main Steam Valve MS-V-15A without removing it from service. The Operations department was contacted to review the valve lubrication program and ensure that similar valves were adequately greased.

# OPERATING DATA REPORT

## OPERATING STATUS

DOCKET NO. 50-289  
 DATE May 14, 1991  
 COMPLETED BY W G HEYSEK  
 TELEPHONE (717) 948-8191

1. UNIT NAME: THREE MILE ISLAND UNIT 1
2. REPORTING PERIOD: APRIL 1991
3. LICENSED THERMAL POWER: 2568
4. NAMEPLATE RATING (GROSS MWe): 871
5. DESIGN ELECTRICAL RATING (NET MWe): 819
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 856
7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 808

### NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):
10. REASONS FOR RESTRICTIONS, IF ANY:

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	719.0	2879.0	146040.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	719.0	2879.0	72043.2
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2245.6
14. HOURS GENERATOR ON-LINE	(HRS)	719.0	2879.0	70998.7
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1714602	6849780	173320112
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	574006	2321464	58282368
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	540107	2184704	54674124
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	48.6
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	48.6
21. UNIT CAPACITY FACTOR (USING MDC NET)		93.0	93.9	47.7
22. UNIT CAPACITY FACTOR (USING DER NET)		91.7	92.7	45.7
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	46.0

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):  
 Cycle 9 Refueling Outage, October 4, 1991, 50 day duration.

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289  
UNIT TMI-1  
DATE May 14, 1991  
COMPLETED BY W G HEYSEK  
TELEPHONE (717) 948-8191

MONTH: APRIL

DAY AVERAGE DAILY POWER LEVEL  
(MWe-NET)

1	762
2	762
3	761
4	758
5	755
6	748
7	743
8	742
9	738
10	745
11	752
12	757
13	758
14	756
15	757
16	750

DAY AVERAGE DAILY POWER LEVEL  
(MWe-NET)

17	749
18	754
19	755
20	754
21	755
22	754
23	751
24	750
25	748
26	747
27	743
28	741
29	750
30	742
31	NA

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April 1991

DOCKET NO. 50-289  
 UNIT NAME TMI-1  
 DATE May 14, 1991  
 COMPLETED BY W. G. Heysek  
 TELEPHONE (717) 948-8191

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report#	System Code 4 & 6	Component Code 5 & 6	Cause & Corrective Action to Prevent Recurrence
						NONE			

<sup>1</sup>  
 F Forced  
 S Scheduled

<sup>2</sup>  
 Reason  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & Licensing Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method  
 1-Manual  
 2-Manual Scram  
 3-Automatic Scram  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for  
 preparation of Data Entry Sheets  
 for Licensee Event Report (LER)  
 File (NUREG-0161)

<sup>5</sup> Exhibit 1 same source

<sup>6</sup> Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **October 4, 1991 (9R)**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **No.**

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? **No.**

If no such review has taken place, when is it scheduled? **3/1/91.**

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

**None planned.**

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

**GPU Nuclear plans to install four Westinghouse Lead Test Assemblies during the reload of the TMI-1 core for cycle 9 operation. Westinghouse fuel technology will be utilized to the extent possible.**

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 441
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

**The present licensed capacity is 752. Planning to increase licensed capacity through fuel pool reracking is in progress.**

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

**1991 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).**